

BRACHYTHERAPY

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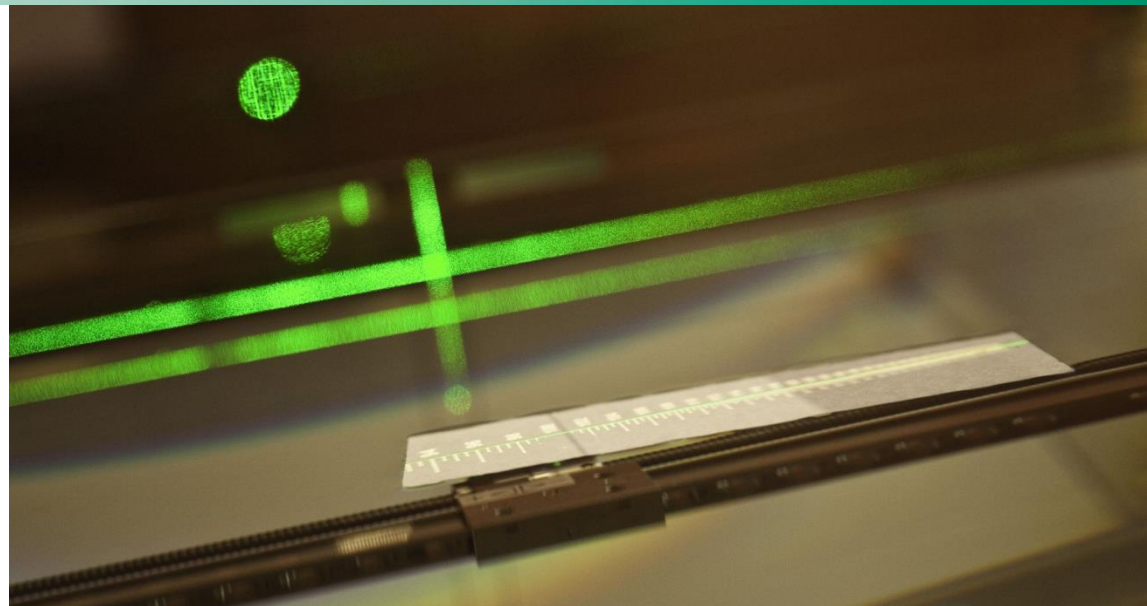
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BERN**



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Overview

- Introduction
- Isotopes
- Techniques
- Planning Techniques
- Dose Calculation

www.estro.org/

The GEC ESTRO Handbook of Brachytherapy

Editors

Erik Van Limbergen
Richard Pötter
Peter Hoskin
Dimos Baltas

ESTRO

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Chapter 3: Radiation protection - *2nd edition 2014*

Chapter 4: Brachytherapy equipment and quality assurance

Chapter 5: Radiobiology of LDR, HDR, PDR and VLDR brachytherapy - *2nd edition 2015*

Chapter 6: Modern imaging in brachytherapy - *1st edition 2002*

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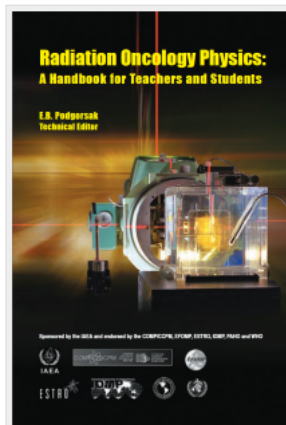
Chapter 8: Treatment planning and evaluation

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www-naweb.iaea.org/nahu/DMRP/RadiationOncologyPhysicsHandbook.html

Dosimetry and Medical Radiation Physics (DMRP)

Radiation Oncology Physics Handbook



The IAEA has published "Radiation Oncology Physics: a handbook for teachers and students" aiming at providing the minimum level of knowledge expected of a medical physicist specializing in radiation therapy.

As a complement to the publication, a set of slides following closely the material in the book has been developed. The slides are designed to be useful to IAEA experts, as teaching material during training events, for students engaged in self-directed studies and for teachers and other interested professionals.

Please, let us know if you discover any errors in the handbook or slides, or if you have suggestions as to the appropriateness of the content or its level. Please, respond in writing to dosimetry@iaea.org.

[Download PDF version of the book \(19.04 MB\)](#)

[Download the Powerpoint version of the slides](#)

Set of accompanying slides for the handbook

- Chapter 1: Basic Radiation Physics
- Chapter 2: Dosimetric Principles, Quantities and Units
- Chapter 3: Radiation Dosimeters
- Chapter 4: Radiation Monitoring Instruments
- Chapter 5: Treatment Machines for External Beam Radiotherapy
- Chapter 6: External Photon Beams: Physical Aspects
- Chapter 7: Clinical Treatment Planning in External Photon Beam Radiotherapy
- Chapter 8: Electron Beams: Physical and Clinical Aspects
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- Chapter 13: Brachytherapy: Physical and Clinical Aspects**
- Chapter 14: Basic Radiobiology
- Chapter 15: Special Procedures and Techniques in Radiotherapy
- Chapter 16: Radiation Protection and Safety in Radiotherapy

Chapter 13

Brachytherapy: Physical and Clinical Aspects

This set of 163 slides is based on Chapter 13 authored by N. Suntharalingam, E.B. Podgorsak, H. Tolli of the IAEA publication (ISBN 92-0-107304-6):

Radiation Oncology Physics: A Handbook for Teachers and Students

Objective:

To familiarize students with basic physical and clinical principles of brachytherapy.



Slide set prepared in 2006 (updated Aug2007)
by E.B. Podgorsak (McGill University, Montreal)
Comments to S. Vatnitsky:
dosimetry@iaea.org

Introduction

Brachytherapy

Greek = close, short

treatment

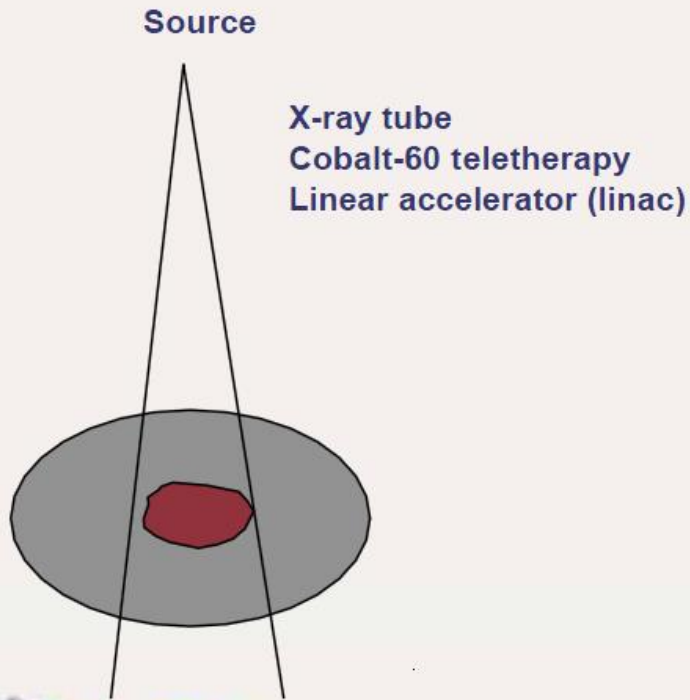
Radiotherapy form in which a sealed radiation source is placed within or near to the region to be irradiated in the body:

- Radiation affects only a very limited area around the radiation source.
- Radiation exposure for healthy tissue located far away from the radiation sources is greatly reduced.
- Hypofractionation (higher single doses, several times daily)
- Radiation sources retain their exact position with respect to the tumor, even if the patient is moving or if there is a displacement of the tumor within the body during the treatment.

13.1 INTRODUCTION

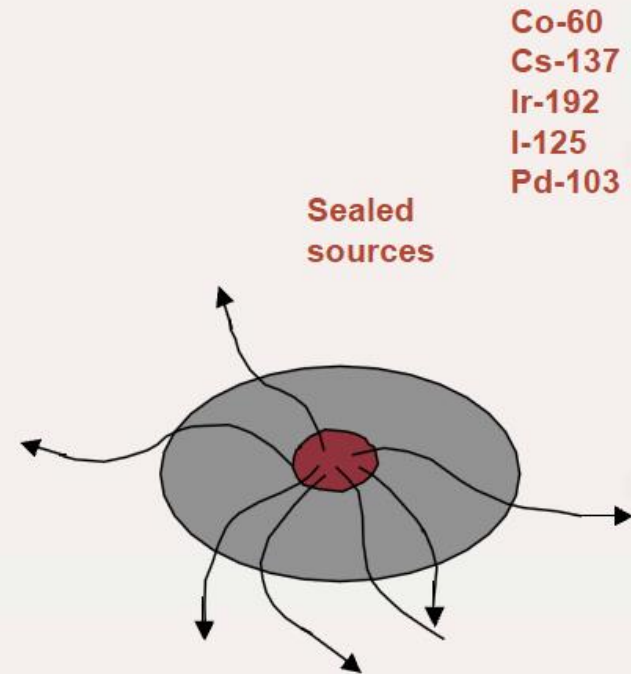
External beam radiotherapy

(external source of radiation)

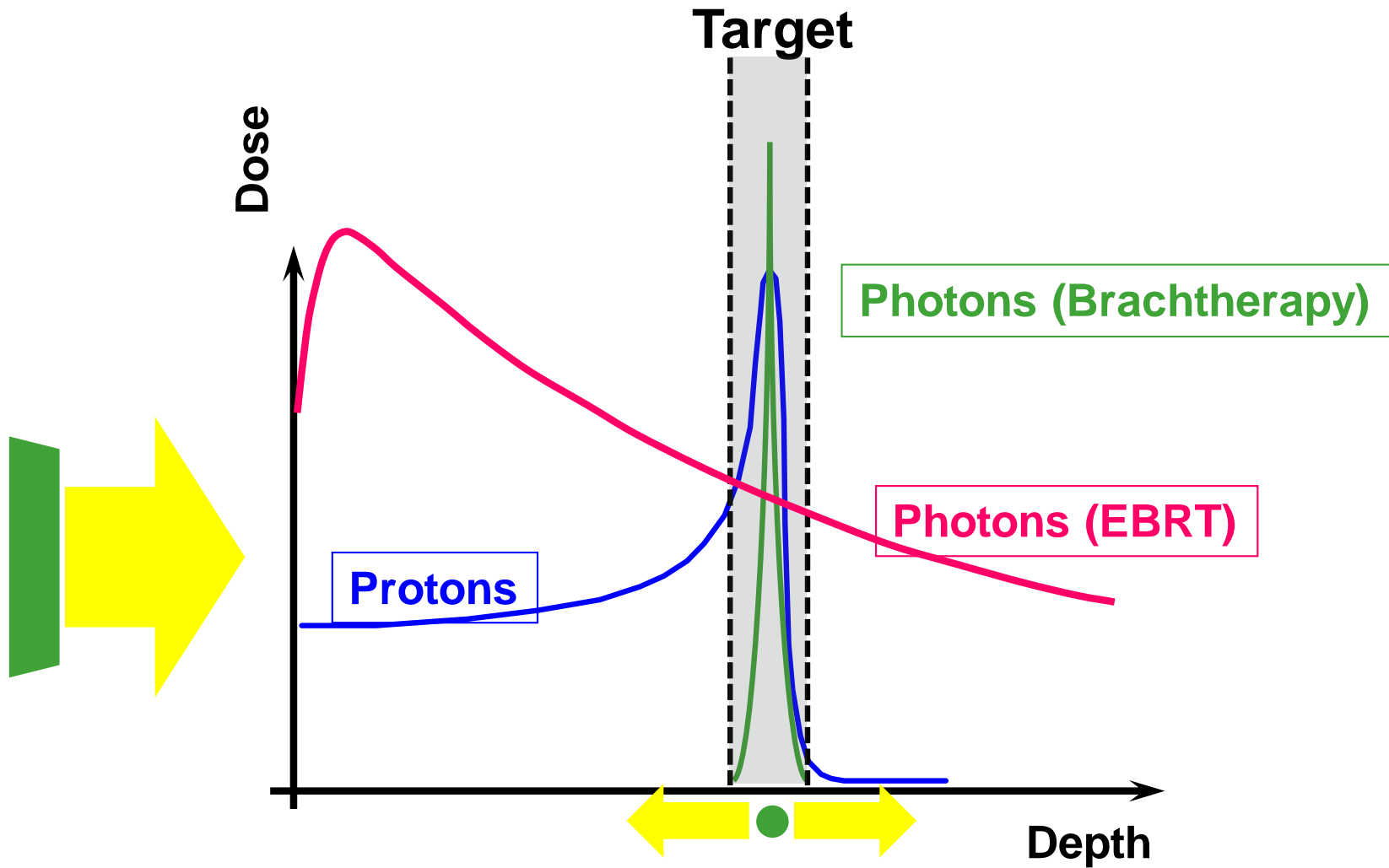


Brachytherapy

(internal source of radiation)



Introduction – Photons vs Protons



Introduction – Brachytherapy vs EBRT

Advantages of Brachytherapy

- Highly localized dose delivery to the target
- Sharp dose fall-off outside the target with good sparing of the surrounding normal tissue leading to less side effects
- Very conformal form of radiotherapy
- Reduction of inter-fraction setup variability
- Shorter treatment time

Disadvantages of Brachytherapy

- For well localized and small tumors
- Team work needed
- Skills in interventional brachytherapy
- Invasive

Introduction – Types of Brachytherapy

Brachytherapy can be categorized in many ways:

- Type of emission
- Duration of the implant
- Location of the implant
- Dose rate
- Type of loading

Brachytherapy sources – Type of emission

Photon sources

- Emission of γ rays through γ decay and/or characteristic x-rays through electron capture and internal conversion. Examples: ^{60}Co , ^{137}Cs , ^{192}Ir , ^{125}I , ^{103}Pd , etc.

Beta sources

- Emission of electrons through beta source decay. Examples: $^{90}\text{Sr}/^{90}\text{Y}$, ^{106}Ru , etc.

Alpha sources

- Emission of α particle through α decay. Example: ^{224}Ra

Neutron sources

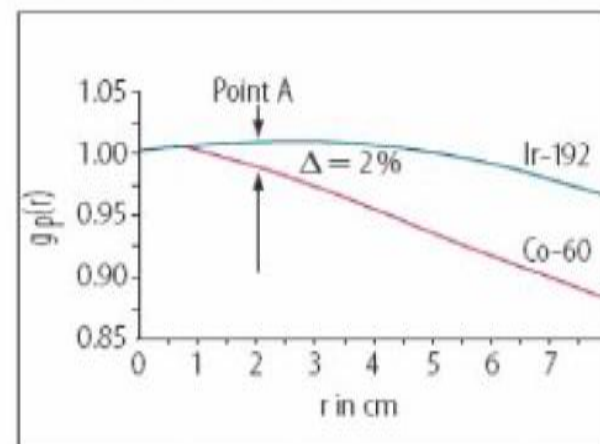
- Emission of neutrons following spontaneous nuclear fission. Example: ^{252}Cf

Miniature x-ray sources

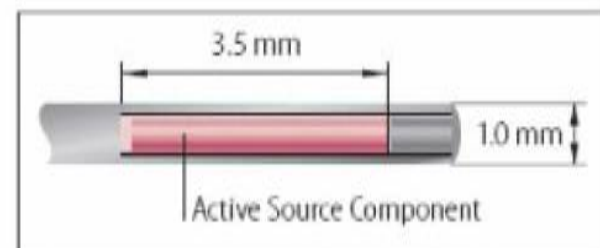
Photon Source Characteristics – Physics

The dose delivered with brachytherapy sources depends on:

- Photon energy spectrum
- HVL
- Half-life
- Specific activity
- Source strength
- Inverse square law

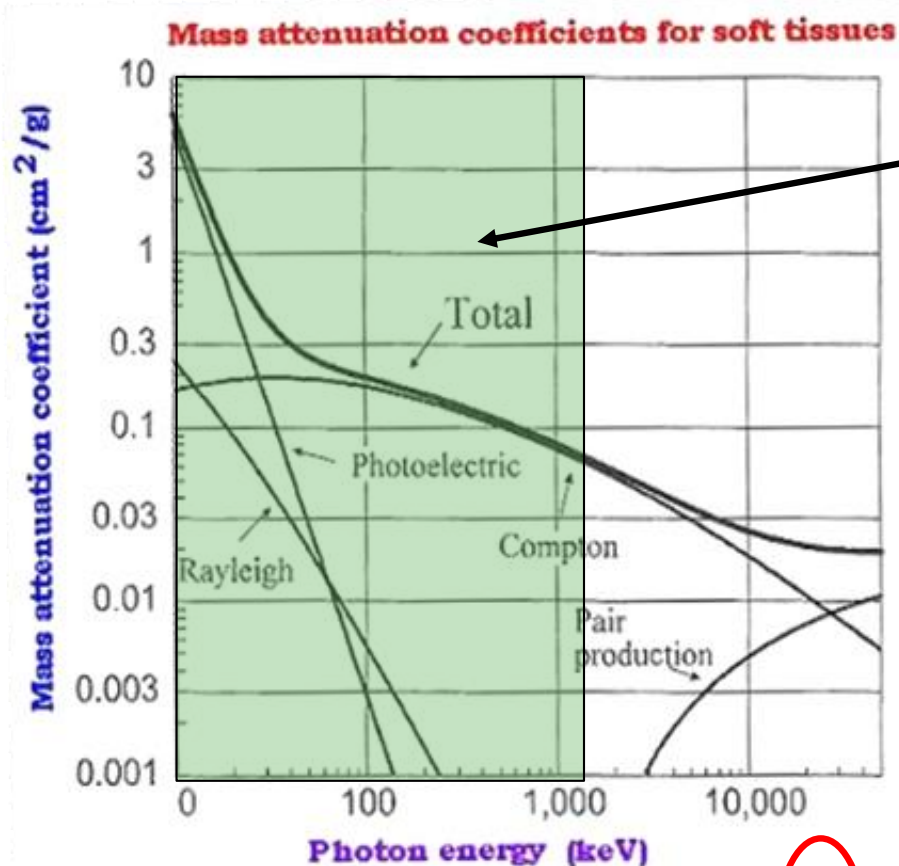


Radial Dose Rate Function



Miniaturised Co-60 Source for HDR afterloading

Photon Source Characteristics – Physics



Brachytherapy

Mass attenuation coefficient: $\frac{\mu}{\rho} = \frac{\tau}{\rho} + \frac{\sigma}{\rho} + \frac{\kappa}{\rho} + \frac{\mu_{el}}{\rho}$

Photoelectric effect ($\tau_{photo} \sim Z^{-4} E_y^{-3}$)

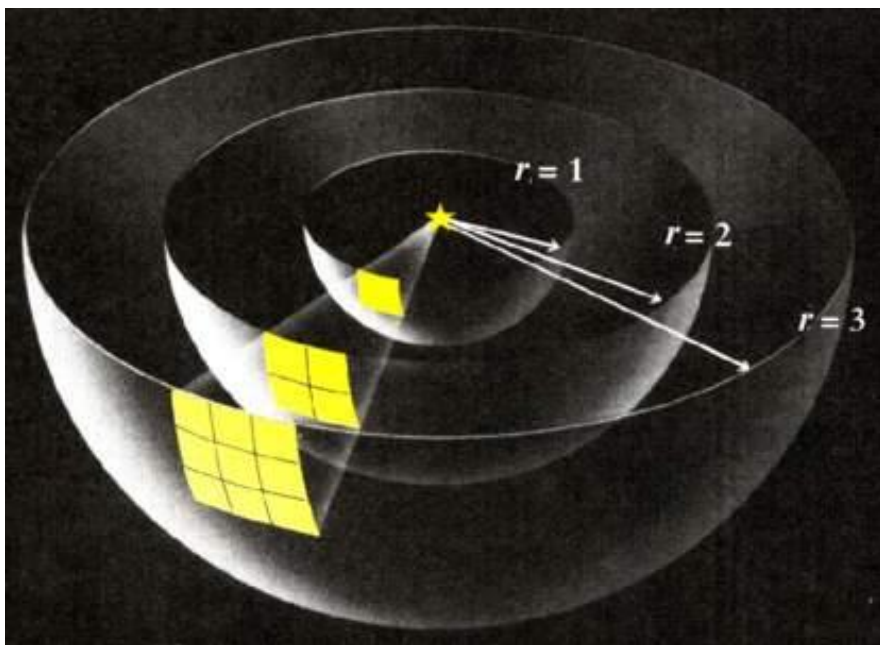
Pair production

Compton effect

Elastic scattering

Photon Source Characteristics – Physics

- Photon Energy of the brachytherapy source influences:
 - Penetration into tissue
 - Radiation protection requirements
- Inverse-square law



Photon Source Characteristics – Radioactive decay

$$N(t) = N_0 e^{-\lambda t}$$

where:

- N_0 is the initial number of radioactive atoms
- N is the number of radioactive atoms at time t
- λ is the decay constant

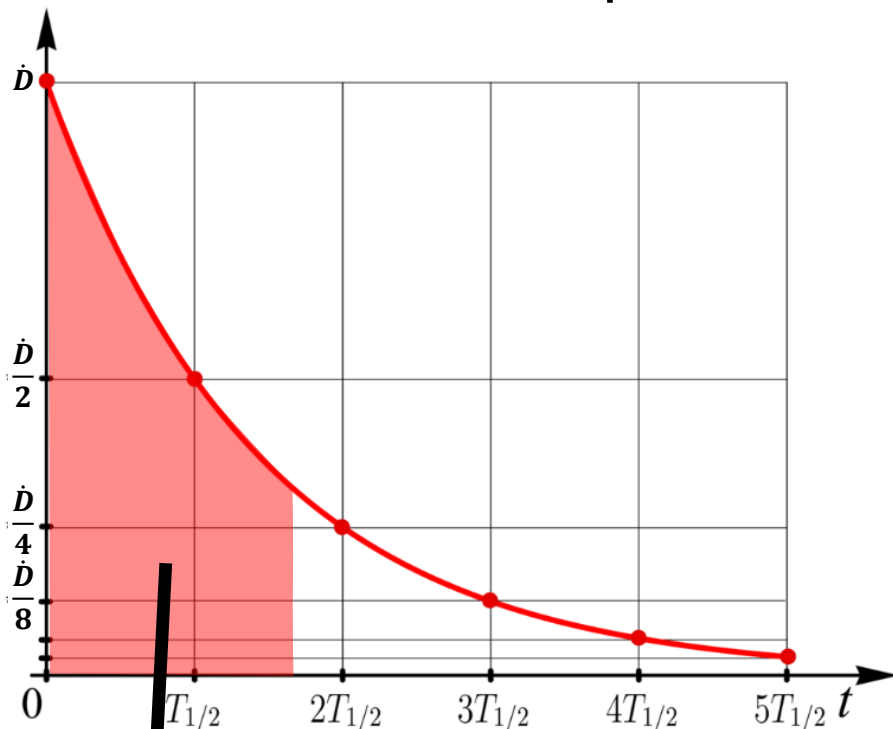
$$\lambda = \frac{\ln 2}{t_{1/2}}, \text{ and } t_{1/2} \text{ is the half-life of the radionuclide}$$

Further, the mean-life T_{avg} of an isotope is defined as the time taken to decay to $1/e$ of the original number of atoms:

$$\frac{N(t)}{N_0} = e^{-1} \Rightarrow \lambda t = 1 \Rightarrow T_{avg} = \frac{T_{1/2}}{\ln 2} = 1.44 * T_{1/2}$$

Cumulative Dose

In calculating the total dose delivered during the implant one must consider the exponential decay of the source strength.



$$D_{cum}(t) = \int_0^t \dot{D}(t) dt = \dot{D}_0 \int_0^t e^{-\lambda t} dt = \frac{\dot{D}_0}{\lambda} \{1 - e^{-\lambda t}\} = \frac{T_{1/2} \dot{D}_0}{\ln(2)} \left\{1 - e^{-\frac{t \cdot \ln(2)}{T_{1/2}}}\right\}$$

Cumulative Dose

$$D_{cum} = \frac{T_{1/2} \dot{D}_o}{\ln(2)} \left\{ 1 - e^{-\frac{t * \ln(2)}{T_{1/2}}} \right\}$$

Permanent implants: $t \gg T_{1/2}$

$$\rightarrow D_{cum} = \frac{T_{1/2} \dot{D}_o}{\ln(2)} \left\{ 1 - e^{-\frac{t * \ln(2)}{T_{1/2}}} \right\}$$

$$D_{cum} = \frac{T_{1/2} \dot{D}_o}{\ln(2)}$$

Temporary implants: $t \ll T_{1/2}$

$$\rightarrow D_{cum} = \frac{T_{1/2} \dot{D}_o}{\ln(2)} \left\{ 1 - e^{-\frac{t * \ln(2)}{T_{1/2}}} \right\}$$

$$D_{cum} = \dot{D}_o t$$

Photon Sources

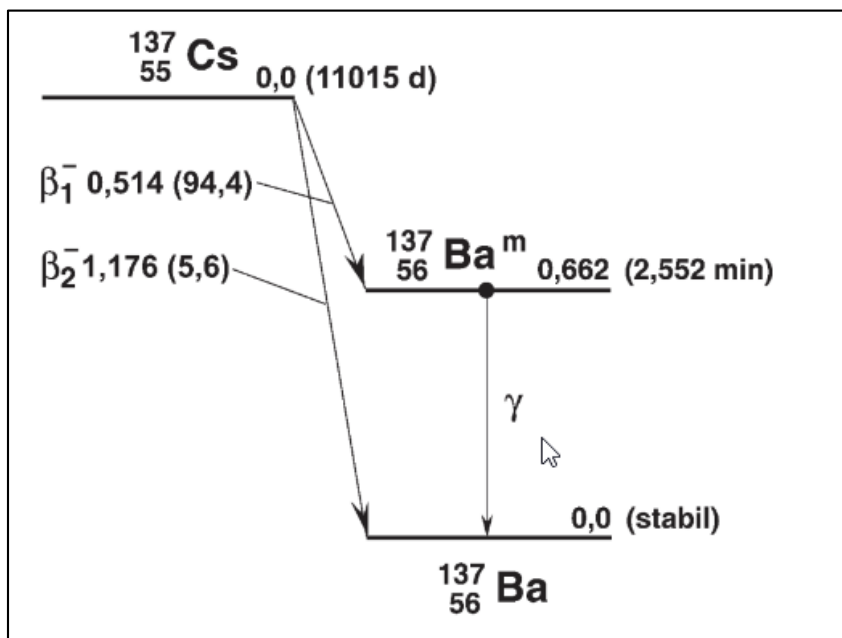
Radionuklid	Energiebereich E (keV)	Mittlere Energie < E > (keV)	HVL _{Pb} (mm)	Halbwertszeit T _{1/2}	Anwendungs- art	A _{spezifische} (GBq/g)
²²⁶ Ra	47-2450	830	8.0	1620y	temporär	37
²⁴¹ Am	-----	60	0.125	432y	temporär	125.8
¹³⁷ Cs	-----	662	5.5	30y	temporär	295.8 x 10 ¹
⁶⁰ Co	1170, 1330	1250	11	5.26y	temporär	40.7 x 10 ³
¹⁹² Ir	136-612	380	2.5	73.9d	temporär	340.4 x 10 ³
¹²⁵ I	27-35	28	0.025	59.6d	permanent	62.9 x 10 ⁴
¹⁶⁹ Yb	10-308	93	0.2	32d	permanent	88.8 x 10 ⁴
¹⁰³ Pd	20-23	21	0.008	17d	permanent	277.5 x 10 ⁴
¹⁹⁸ Au	-----	412	2.5	2.7d	permanent	88.8 x 10 ⁵

Cesium 137

Cesium 137, a fission byproduct, is a popular radium substitute because of its 30-year half-life.

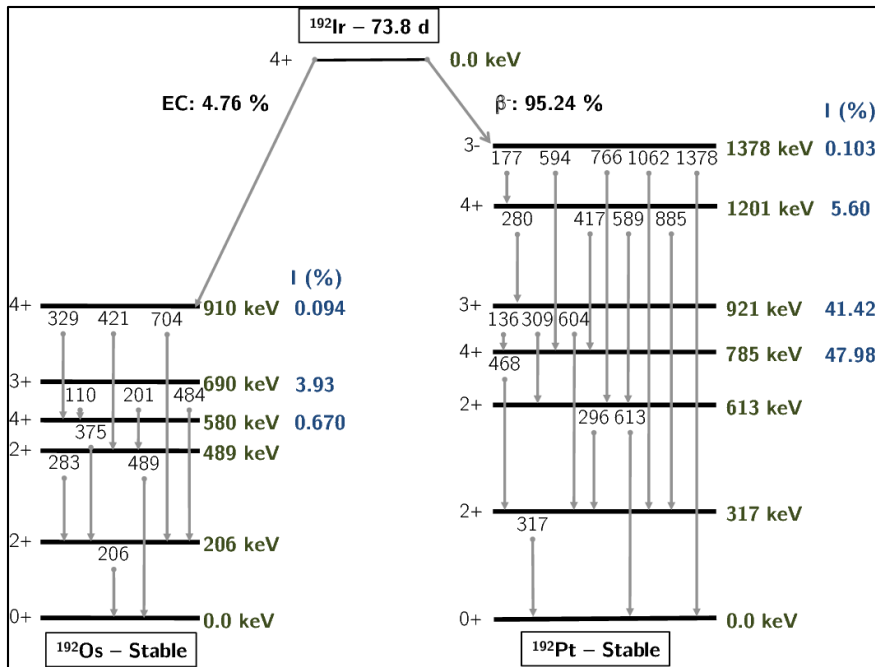
Its single γ -ray (0.66 MeV) is less penetrating ($HVL_{Pb} = 0.65$ cm) than the γ -rays from radium ($HVL_{Pb} = 1.4$ cm) or ^{60}Co ($HVL_{Pb} = 1.1$ cm).

Because ^{137}Cs decays to solid barium 137, ^{137}Cs sources have virtually replaced ^{226}Ra intracavitary tubes in LDR gynecologic applications.



Iridium 192

^{192}Ir is produced in the nuclear reactor in the reaction $^{191}\text{Ir}(n,\gamma)^{192}\text{Ir}$. ^{191}Ir composes 37.3% of natural iridium, ^{193}Ir making 62.7%.



Complex decay pattern leading to a photon spectrum with mean energy of ca. 380 keV

High specific activity \rightarrow small sources

Half life: 73.8 days

Palladium 103

^{103}Pd can be produced:

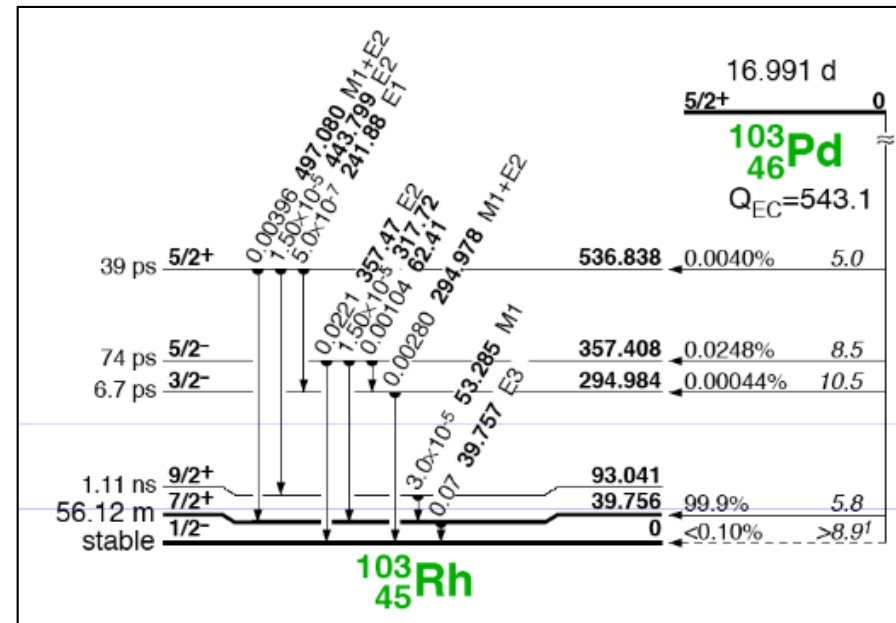
- By neutron activation of ^{102}Pd : $^{102}\text{Pd}(n,\gamma)^{103}\text{Pd}$. (^{102}Pd occurs only at 0.9% level)
- By nuclear reaction with a proton beam on ^{103}Rh :
 $^{103}\text{Rh}(p,n)^{103}\text{Pd}$. (natural abundance of ^{103}Rh : 100%).

In practice, this isotope can be produced with a very high specific activity, more than 2500 GBq/mg.

^{103}Pd decays by electron capture to excited states of Rh-103 followed by characteristic x-ray emission 20-23 keV photons (average 21 keV)

Half-life: 17 days

Widely used for permanent implants



Iodine 125

^{125}I is produced mainly in a neutron capture process (in reactors), through ^{124}Xe gas target: $^{124}\text{Xe}(n, \gamma)^{125}\text{Xe}$.

^{125}Xe decays into ^{125}I by electron-capture (EC) transition: $^{125}\text{Xe} \rightarrow ^{125}\text{I} + \nu + E_b$.

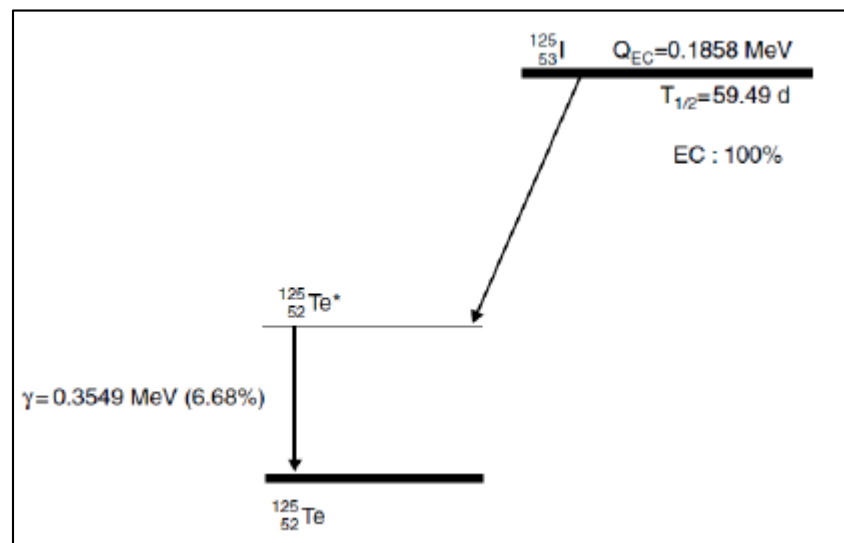
^{125}I decays by EC into an excited state $^{125}\text{Te}^*$, producing the maximum photon energy of 35.5 keV by gamma decay (6.7% of the time).

In addition, the transition leads to characteristic x-rays of energy between 27.2 to 31.7 keV (K-shells) as a result of internal conversion (93.3%).

The specific activity of ^{125}I is more than 600 GBq/mg

Half life: 59.4 days

Iodine seeds are widely used for permanent implants (prostate seed implants) and also eye plaques.



Photon Source Characteristics

High Dose Rate / Brachytherapy Sources

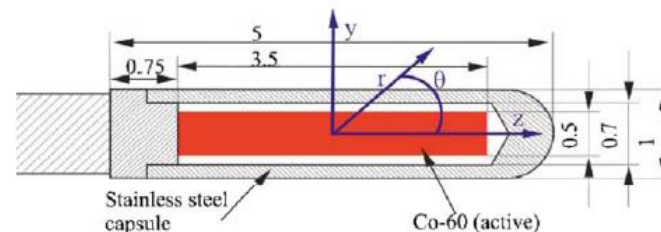
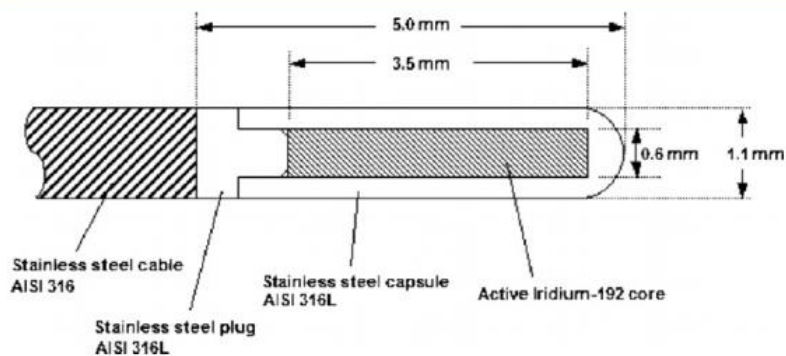


Figure 1. Schematic drawing of the Nucletron 'Classic' ^{192}Ir HDR brachytherapy source.

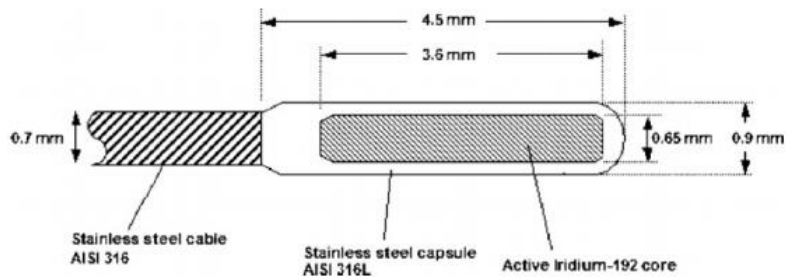
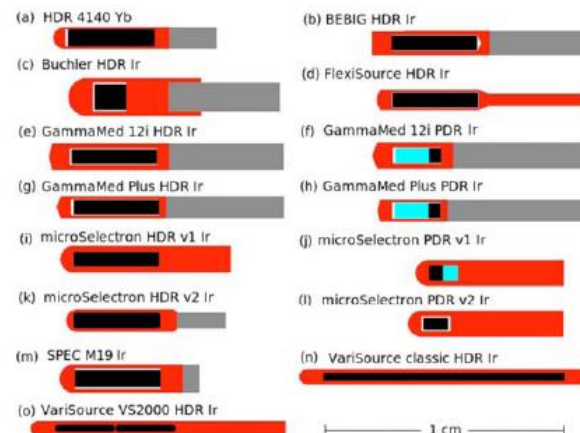
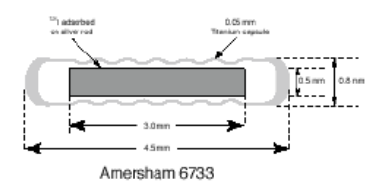
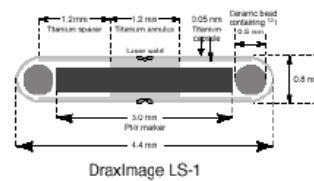
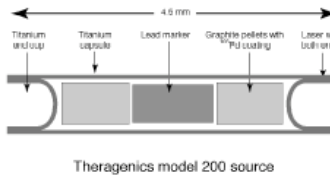
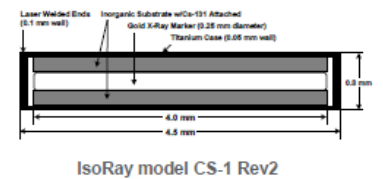
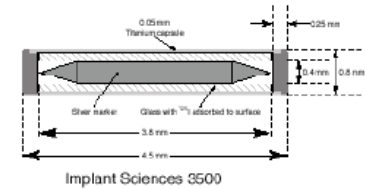
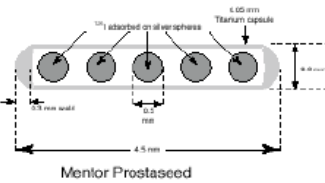
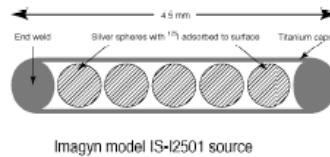
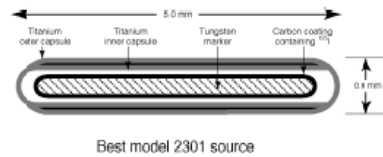
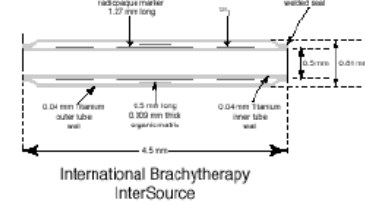
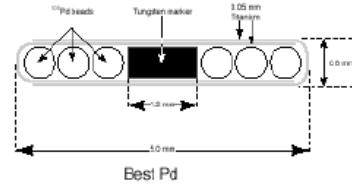
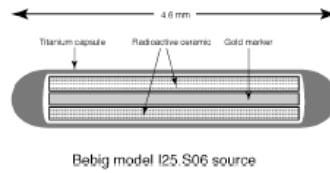
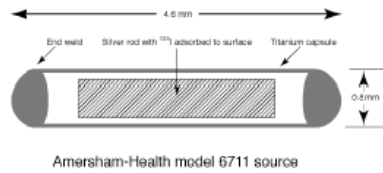
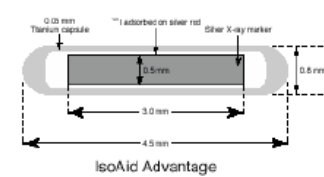
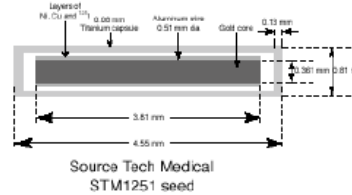
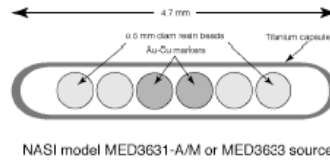
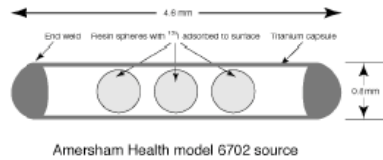


Figure 2. Schematic drawing of the Nucletron 'V2' ^{192}Ir HDR brachytherapy source.



Photon Source Characteristics

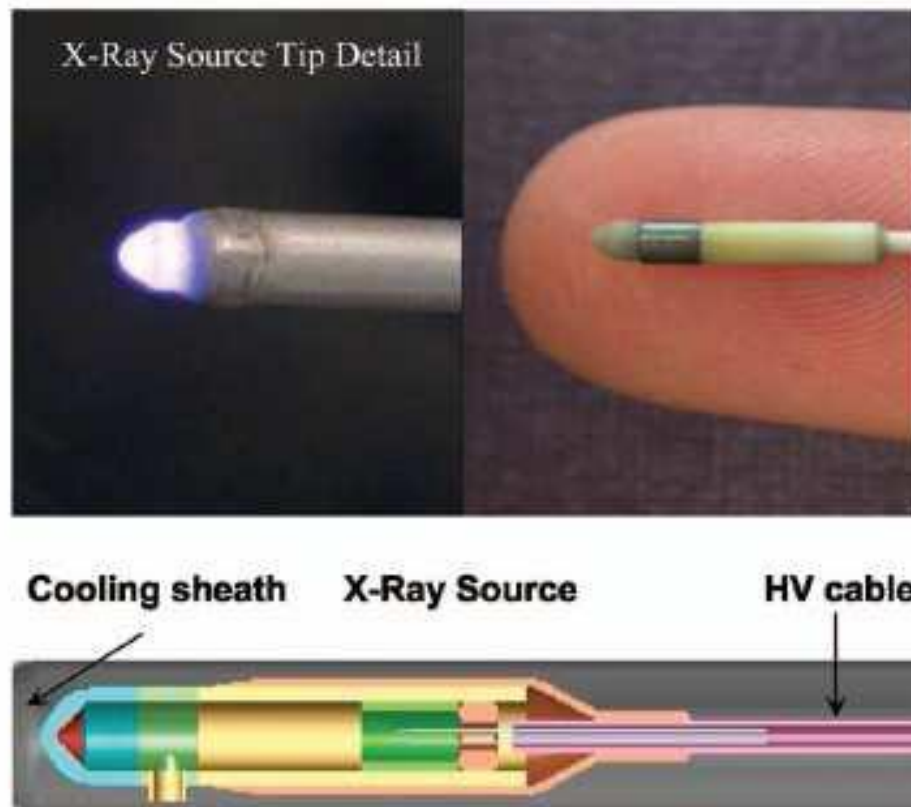
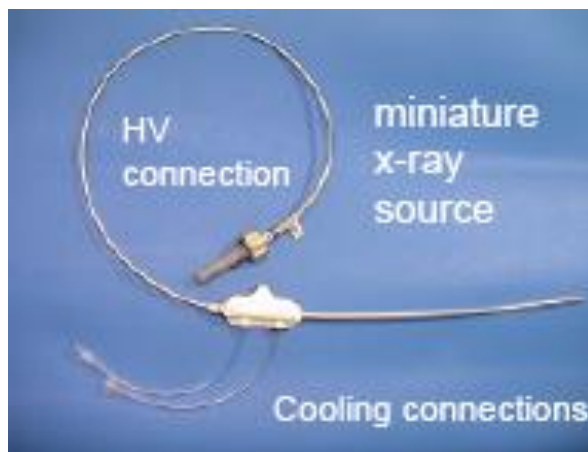
Low Dose Rate / Brachytherapy Sources



Photon Source Characteristics

Electronic X-Ray Source

- 50 kV, 0.3 mA, 15 W
- Water cooled
- Can be used as HDR source



Rivard et al, Med Phys. 33(11), 2006

Photon Source Characteristics – Source Specification

Activity

Historic units:

- For Radium: [mg Ra] → «Dose» specification in [milligram-hours]
- For other nuclides: [mg Ra equivalent]
- $1 \text{ [Ci]} = 1 \text{ [g Ra]} = 3.7 * 10^{10} \text{ [disintegration / sec]}$

SI Unit

- $1 \text{ [Bq]} = 1 \text{ [disintegration / sec]} \rightarrow 1 \text{ [Ci]} = 3.7 * 10^{10} \text{ [Bq]}$

Photon Source Characteristics – Source Specification

The amount of radiation emitted depends on the source geometry (filtration and self absorption)

Specification of source strength as “activity”

- Difficult to measure accurately and reproducibly both by the vendor and the user
- Variability in the factor to convert activity to dose in the patient

Photon Source Characteristics – Source Specification

Specification of gamma sources:

- Reference air kerma rate in air: $(\dot{K}_{air}(d_{ref}))_{air}$
 Defined by the ICRU reports 38 and 58 as the air kerma rate in air at the reference distance $d_{ref} = 1\text{ m}$, corrected for the air attenuation and scattering [$\mu\text{Gy} / \text{h}$]
- Air kerma strength: $S_k = (\dot{K}_{air}(d_{ref}))_{air} \times d_{ref}^2$
 In the AAPM TG43 recommendations: $1\text{ U} = 1\text{ cGy} \cdot \text{cm}^2 \cdot \text{h}^{-1}$
- Air kerma rate in air: $(\dot{K}_{air}(d))_{air} = \frac{A_{app}\Gamma_{AKR}}{d^2}$
 A_{app} : apparent activity [Bq]
 Γ_{AKR} : air kerma rate constant [$\mu\text{Gy} \cdot \text{m}^2 / (\text{GBq} \cdot \text{h})$]
 d : distance [m]

KERMA: **K**inetic **E**nergy **R**elaxed per unit of **M**ass. Mean energy transferred from indirect ionizing radiation (photons) to charged particles (electron).

Source Strength – Certificate

Certificate For sealed Sources G2-0

Serial Number of Afterloader: _____

Customer Name and Address: **INSELSPITAL**
 FREIBURGSTRASSE 10
 3010 BERN
 SWITZERLAND

Issue Date: 2016-10-20 ⁽¹⁾

Model Designation:	REF	105.002
Serial Number:	SN	D36G2082
Production Code:	LOT	27344/02 (DRN 07736)

Manufacturer Code: NLF 01 ⁽⁷⁾

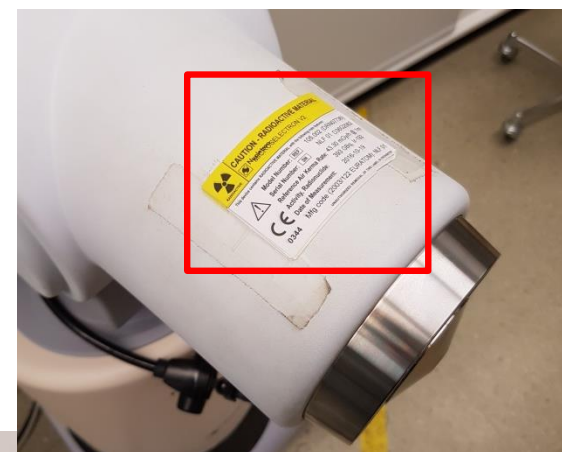
Serial Number of Transport Container: 2551C6
 Serial Number of Check Cable: Not applicable
 Certificate Number: KLbS9 Kv@o+ V&BBn wX@Rd h1

SOURCE SPECIFICATIONS

Reference Air Kerma Rate: 43.31 mGy h⁻¹ +/- 5% at 1 m ⁽²⁾
 Measured at: 2016-10-19 15:33 CET ⁽¹⁾
 Estimated Content Activity: 393.7 GBq (10.64 Ci) at date of measurement ^(3,4)

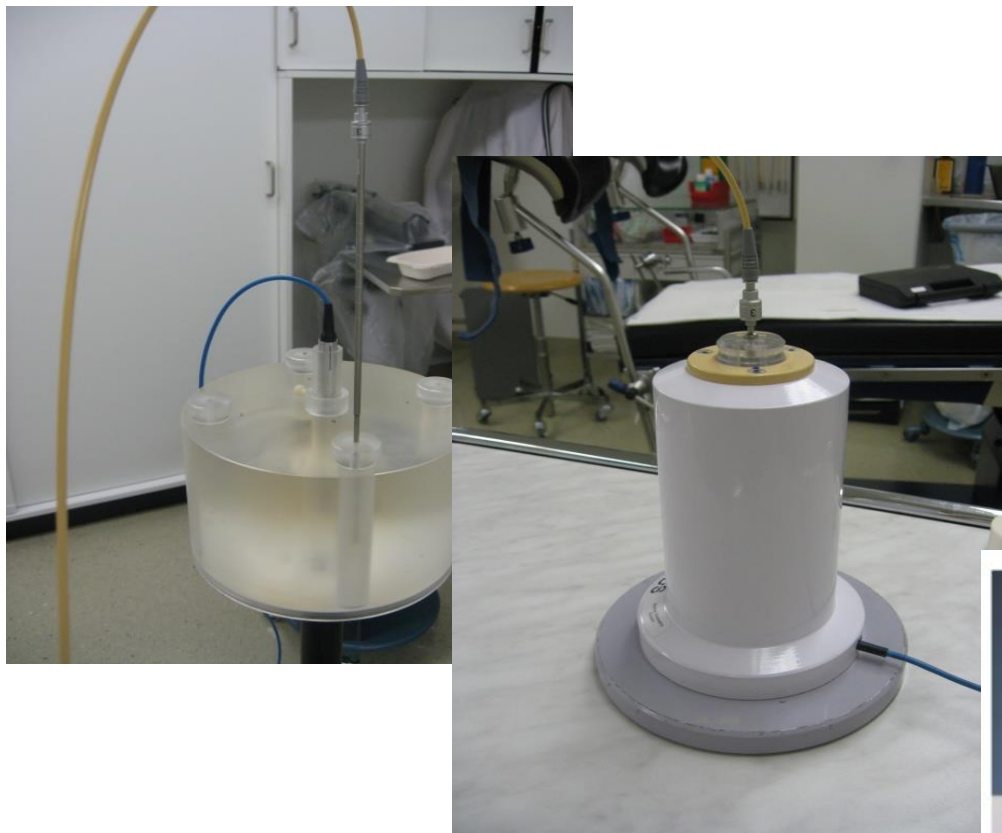
Source Type: MICROSELETRON V2
 Capsule Dimensions: 0.9 mm diameter, 4.5 mm length
 Source Pellet Dimensions: 0.6 mm diameter, 3.5 mm length
 Source Pellet Form: solid iridium
 Radionuclide: 192Ir, gamma radiation source
 Encapsulation: single

All international recommendations indicate the convention of specifying the sources exclusively in units of reference air kerma rate traceable to Accredited Calibration Laboratory.



- (1) Date format yyyy-mm-dd.
 (2) At Confidence level of 99.7%. Air Kerma rate at 1 m from the centre of the source in a radial direction, i.e. perpendicular to the symmetrical axis of the source.
 (3) The estimated content activity is determined by applying a conversion factor (0.110 mGy m² h⁻¹ GBq⁻¹) to the measured gamma radiation output of the sealed source determined with a calibrated instrument.
The instrument is calibrated against the standard of the Physikalisch-Technische Bundesanstalt (PTB), Braunschweig, Germany.
 (4) The estimated content activity is the activity for the specified radionuclide, other radionuclides are not detectable.
 (5) Leakage test method according to ISO9978 method Liquid nitrogen bubble test (6.2.4).
 (6) Surface contamination test in accordance to ISO9978 method Wet Wipe Test (5.3.1).
 (7) European manufacturer's code in accordance with European council directive 2003/122/EURATOM.

Photon Source – Source Strength



Well-Chamber

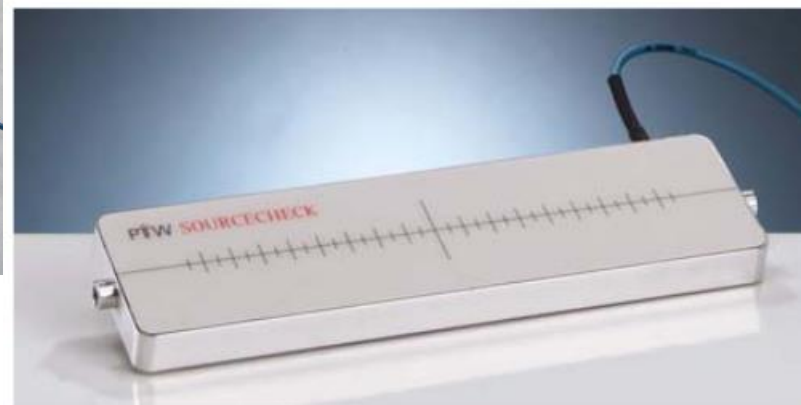
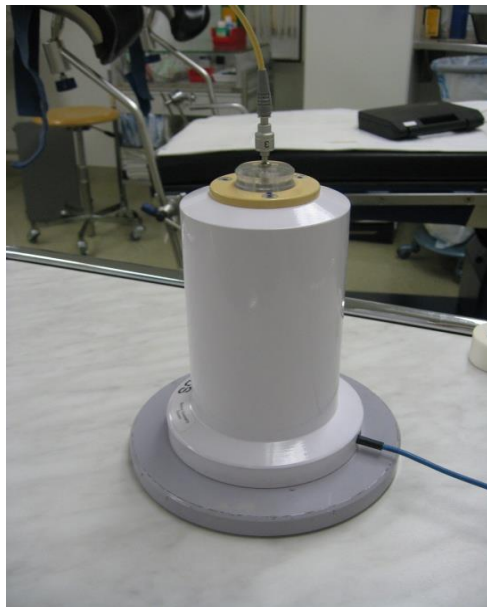


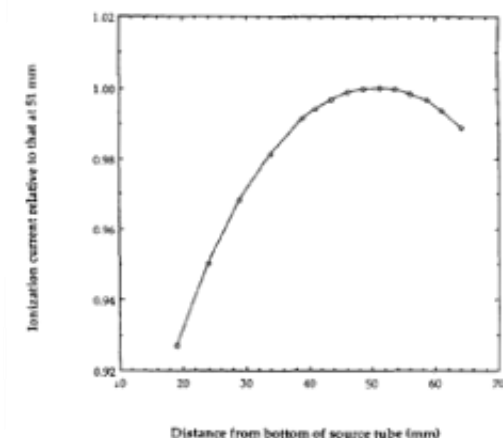
Figure 2: SourceCheck source strength device

Source Strength – Well Chamber



Well chamber and electrometer having calibrations traceable to an Accredited Calibration Laboratory (within 2 years as indicated by the dates on the form)

The electrometer needs to be calibrated in both current and charge (integral) modes.

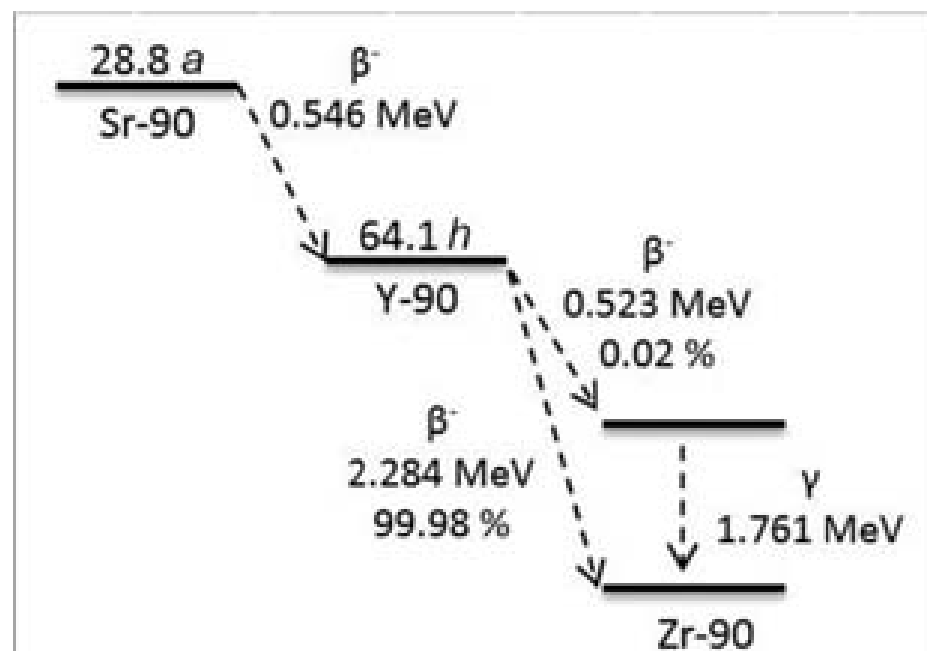


The source is programmed to go to a series of positions within the well chamber and the maximum current reading is used to calculate the activity in air kerma units.

Consistency check every 3 months (with a Cs137 Source)

Beta sources – $^{90}\text{Sr}/^{90}\text{Y}$

- Byproduct of nuclear fission
- Therapeutic radiation is primarily from 2.27 MeV betas from Y-90
- Suitable for treatment of superficial lesions, ocular lesions and coronary vessels
- Limited depth of penetration



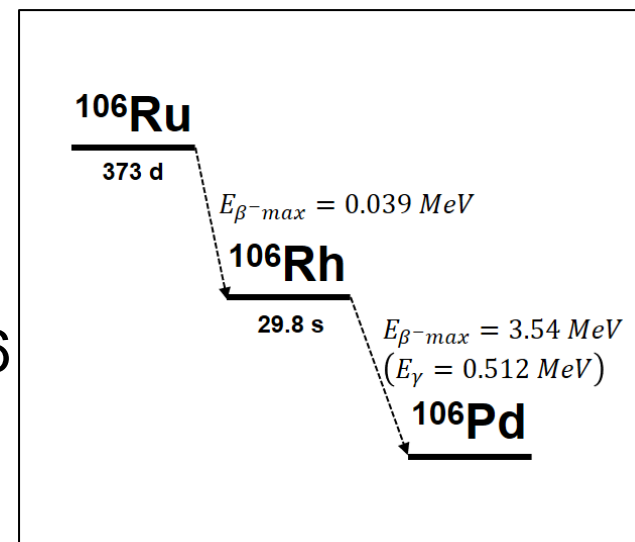
Beta sources – ^{106}Ru

Ruthenium-106 (^{106}Ru) is a pure β^- emitter with a half-life of ~ 373 days.

It decays to the ground state of rhodium-106 (^{106}Rh). The half-life of ^{106}Rh is only ~ 29.8 seconds

Rhodium-106 is also a pure β^- emitter. Its decay is followed by emission of gamma-rays from de-excitations of its daughter-nucleus ^{106}Pd .

^{106}Ru is commercially obtained from neutron irradiated high enrichment ^{235}U target in process of production ^{99}Mo . After isolation of ^{99}Mo radioisotope and decaying of ^{103}Ru , ruthenium is separated from the wastes by multistep procedure.



Brachytherapy – Treatment duration

Temporary implant

- Dose is delivered over a period of time that is short in comparison with the half-life of the sources
- Sources are removed when the prescribed dose has been reached

Permanent implant

- Dose is delivered over the whole lifetime of the sources
- The sources undergo complete radioactive decay
- In general, the sources are not removed from the patient after complete decay

Brachytherapy – Type of implants

Intracavitary

- GYN, rectum

Interstitial

- HNO, lip, eyelid, mamma, rectum, prostate

Surface plaque

- Skin surface or eye cornea

Intraluminal

- Bronchus, oesophagus, HNO

Intraoperative

- Single catheter or flaps for sarcomas, gynecologic and rectal cancer recurrences

Intravascular

- Peripheral vascular diseases (vessels narrowing)

Brachytherapy – Dose rate

- Low dose rate (LDR): 0.4 - 2 Gy/h
- Medium dose rate (MDR): 2 - 12 Gy/h
- High dose rate (HDR): >12 Gy/h
- Pulsed dose rate (PDR)
Simulation of a low dose rate (LDR) treatment (50-100cGy/h) by a series of short dose pulses separated by intervals of 1 hour to several hours

Brachytherapy – Source loading

- Hot loading
The applicator is pre-loaded and contains radioactive sources at time of placement into the patient

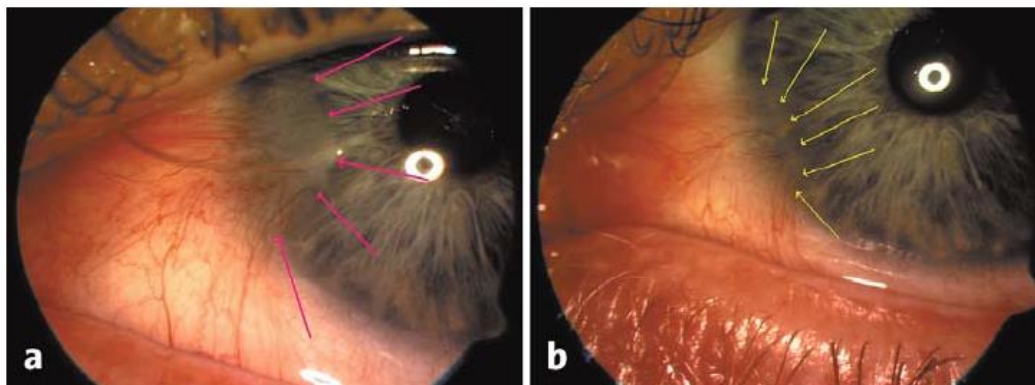
- Afterloading
The applicator is placed first into the patient and the radioactive sources are loaded later:
 - Manual afterloading
 - Automatic remote afterloading

Source loading – Hot loading

- Pterygium is a benign conjunctival neoformation. Standard treatment is a surgical excision followed by adjuvant beta irradiation.
- $^{90}\text{Sr}/^{90}\text{Y}$ eye applicators are used for treatment of pterygium.



Figure 1. Strontium-90/yttrium-90 applicator.

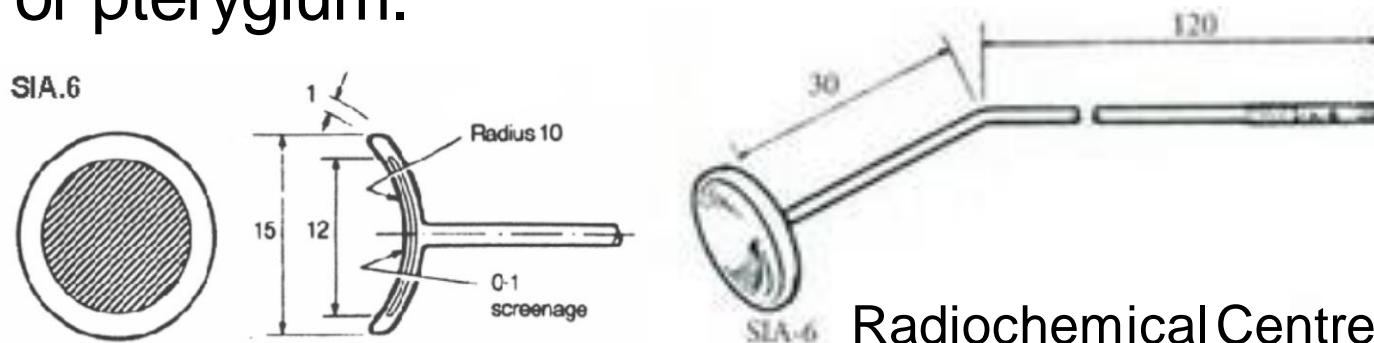


Figures 3a and 3b. A patient's eye prior to (a) and after treatment (b) at 6-month follow-up.

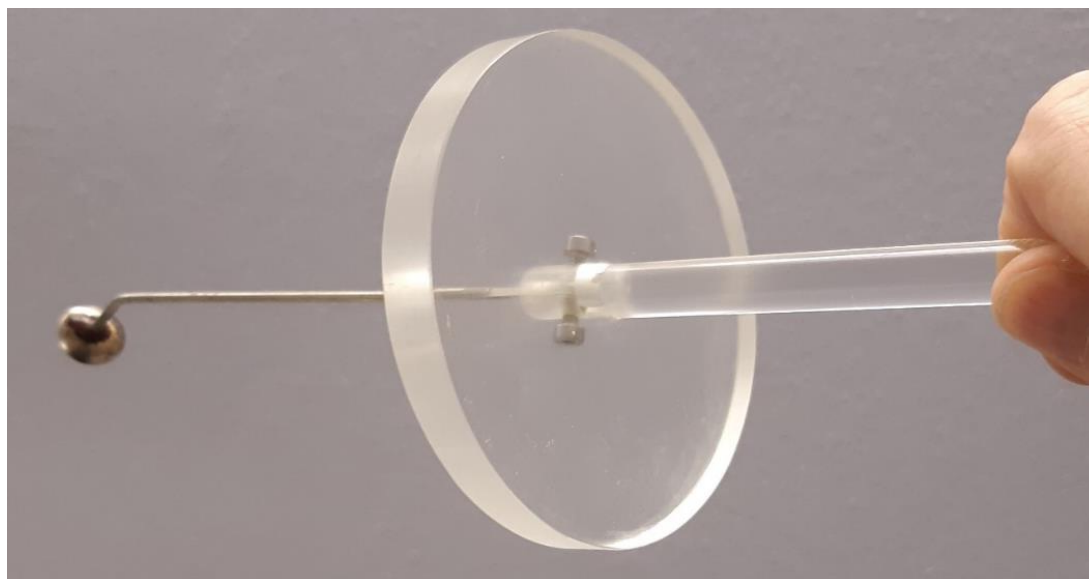
I. Vastardis et al., Strahlenther Onkol 185, 808–14. 2009

Source loading – Hot loading

Curved $^{90}\text{Sr}/^{90}\text{Y}$ eye applicator (SLA.6) used for treatment of pterygium.



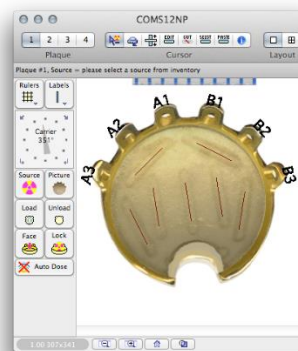
Radiochemical Centre, Amersham, UK



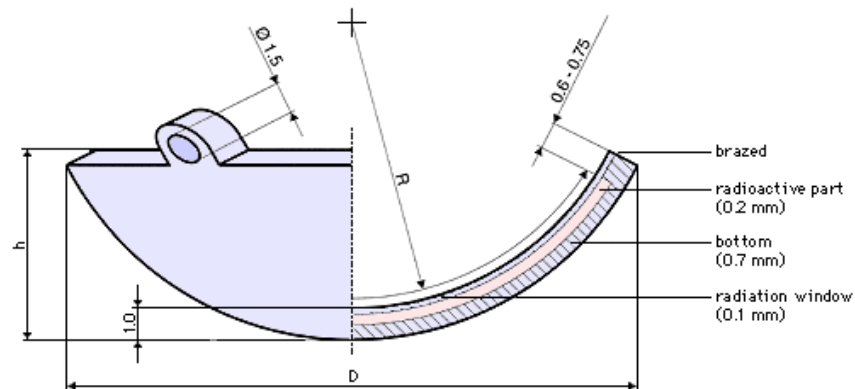
Source loading – Hot loading

Intraocular melanoma is the most malignant eye tumor, which can be treated with eye plaques either:

- loaded with radioactive ^{125}I seeds



- or where a thin film of ^{106}Ru is encapsulated within a sheet of pure silver (Ag 99.99), with a total thickness of 1 mm



From: eyephysics.com

Source loading – Afterloading

Manual Afterloader: Radiation sources are manually afterloaded into applicators or catheters that have been placed within the target volume:

- Manual loading and removal of the sources from the applicators or catheters result in some radiation exposure of the medical and support staff

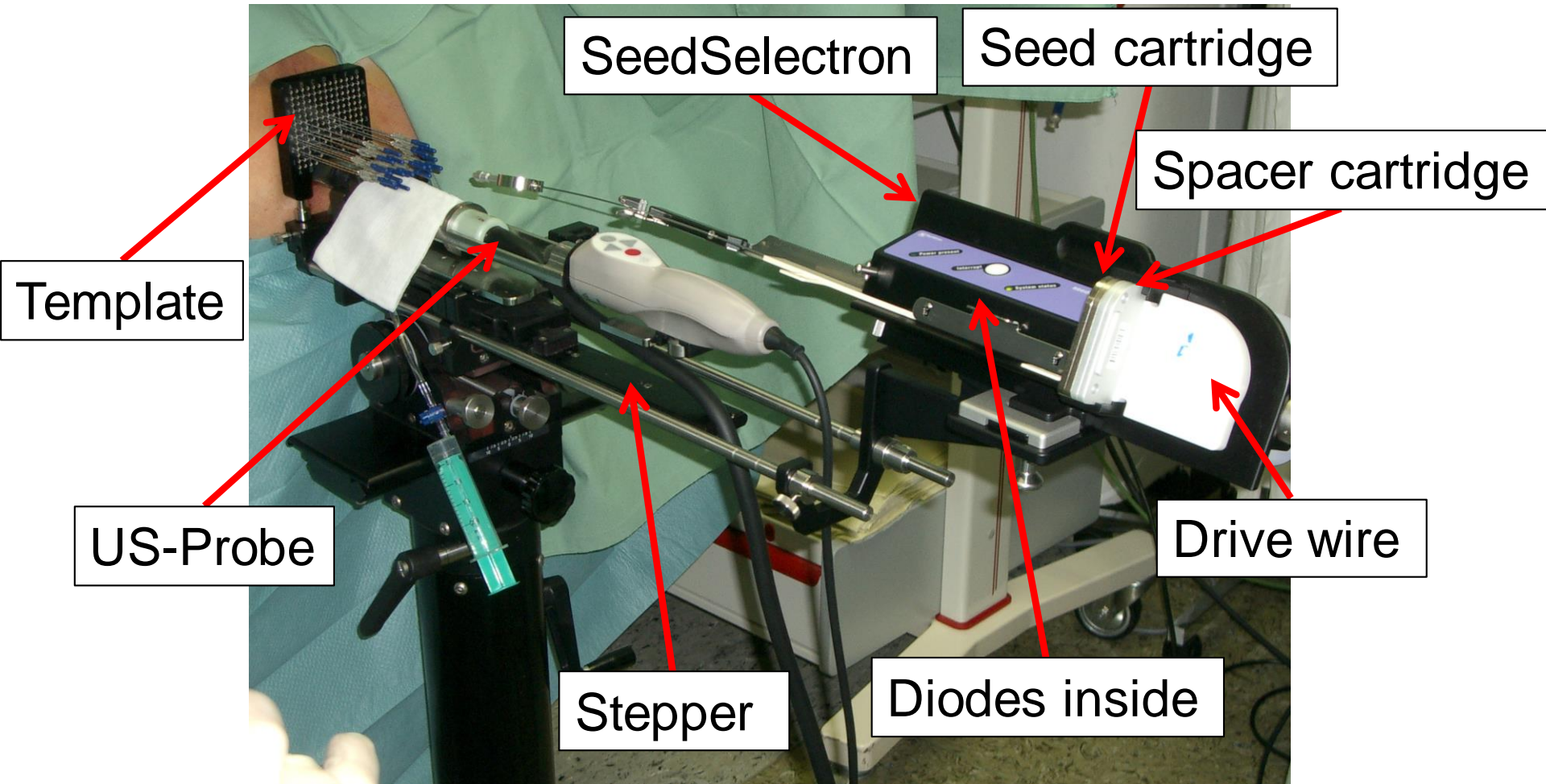
Source loading – Afterloading

Remote Afterloader: Computer driven afterloading systems:

- Reduction of the radiation exposure to medical and support staff
- Increase patient throughput
- Consistent and reproducible treatment delivery

Source loading – LDR Afterloading Systems

Permanent Prostate Implant with ^{125}I seeds



Source loading – HDR Afterloading Systems



Flexitron (Elekta)



MultiSource
(Eckert & Ziegler BEBIG)

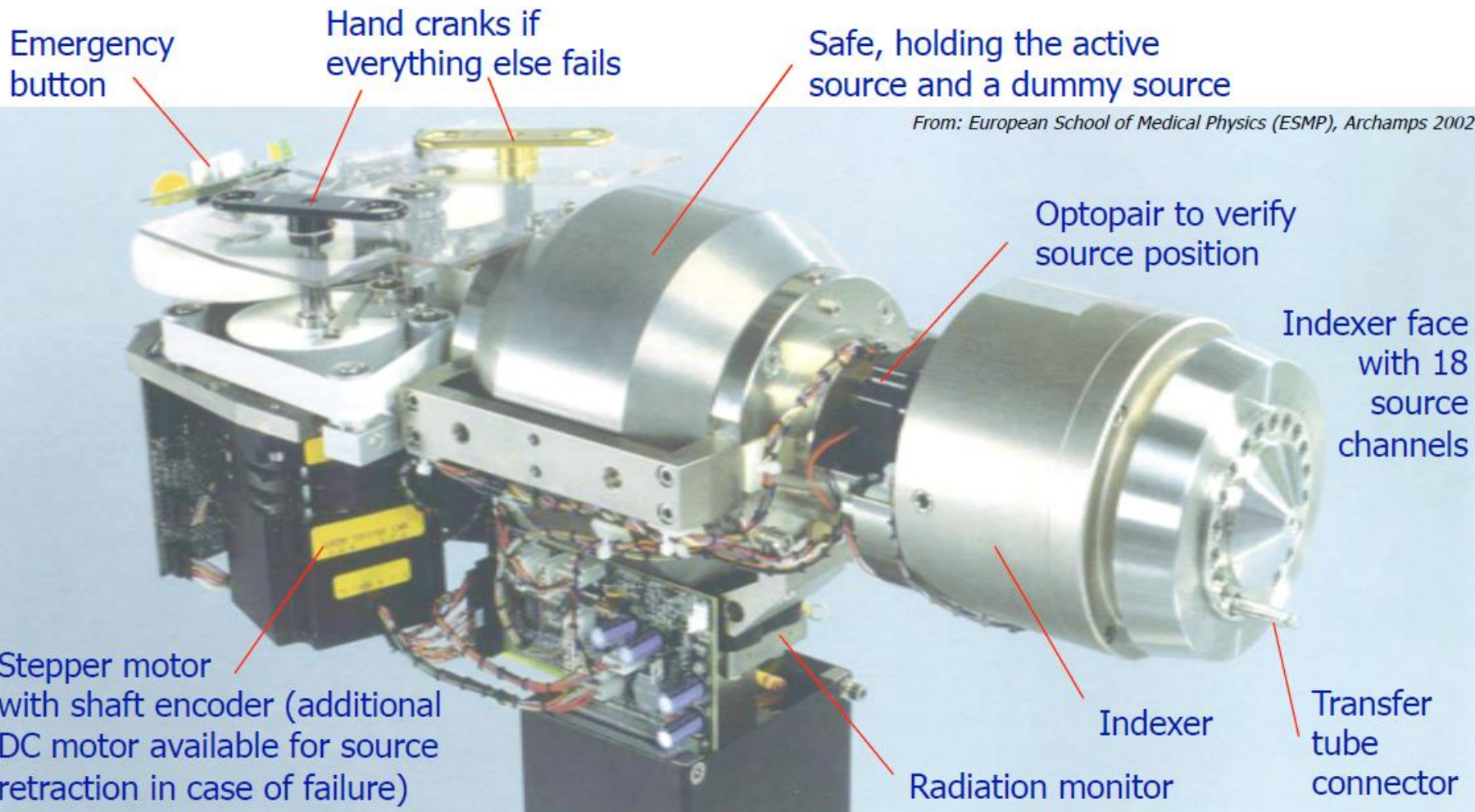


microSelectron (Elekta)

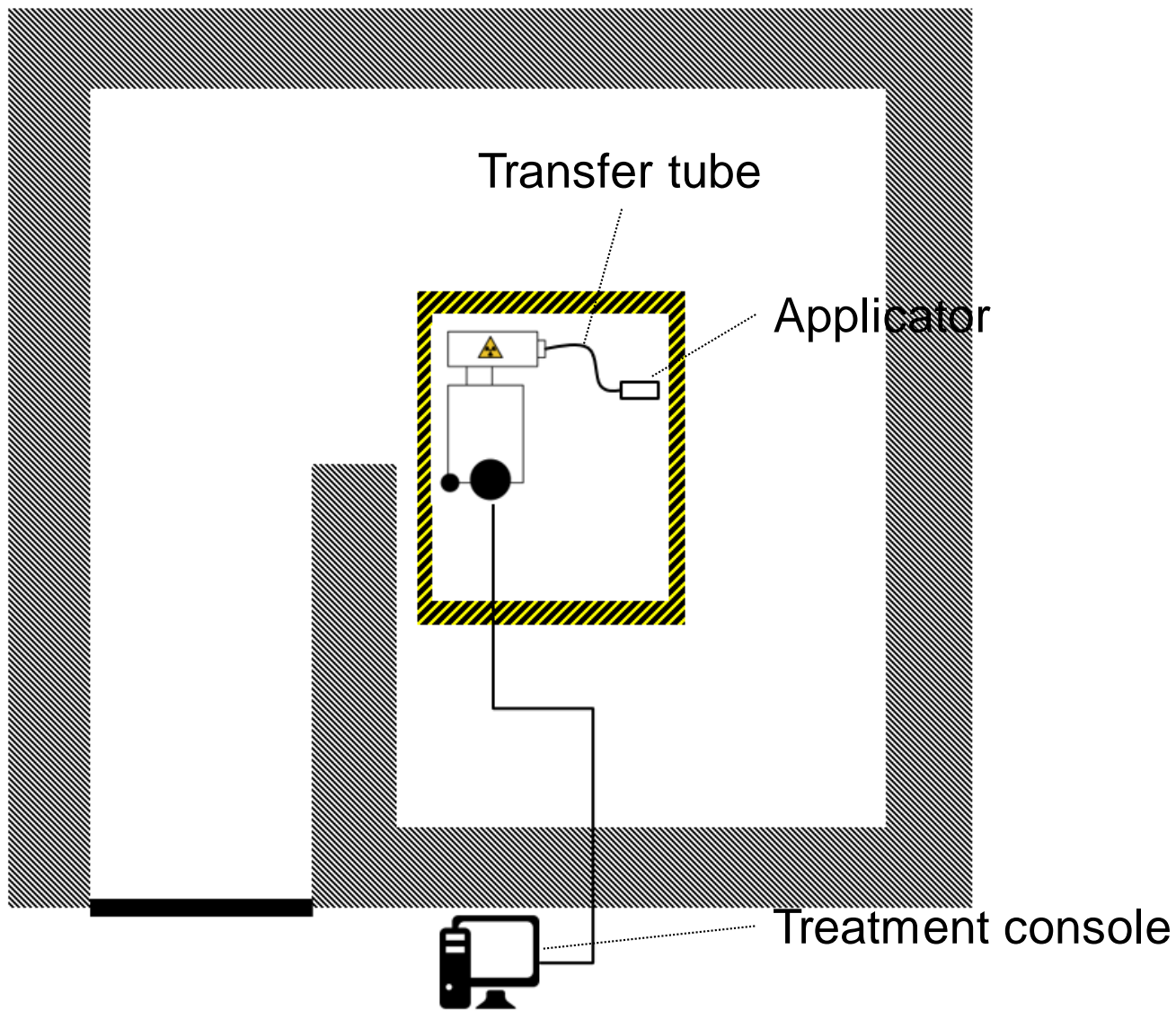


GammaMed (Varian)

Source loading – HDR Afterloading Systems



Brachytherapy – HDR Afterloading Systems



Brachytherapy – HDR Afterloading Systems



Brachytherapy – HDR Afterloading Systems



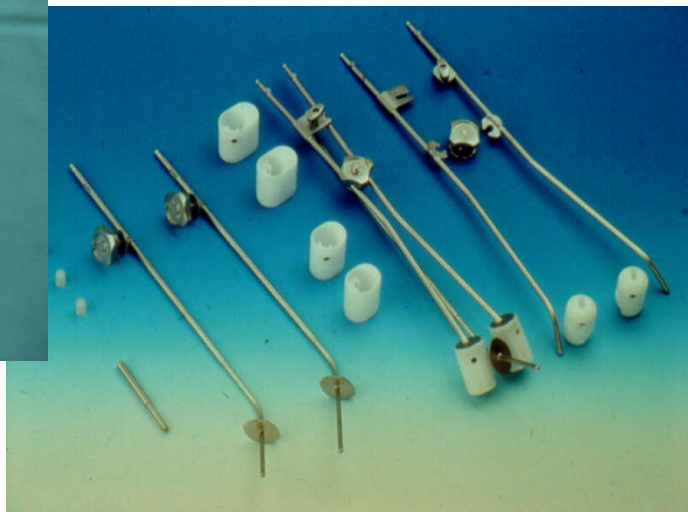
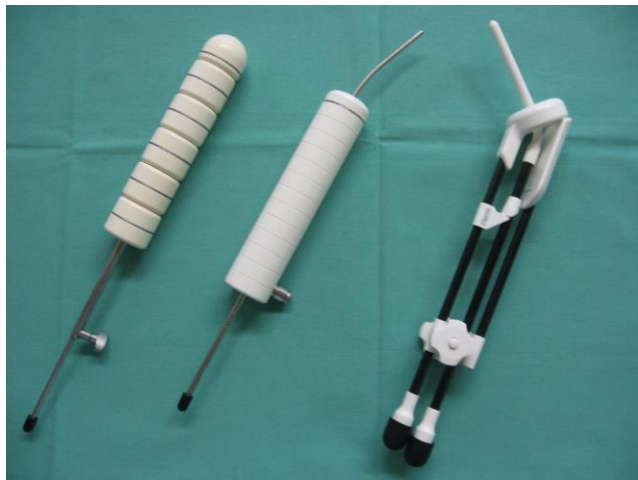
Brachytherapy – Applicators

Surface applicators

Leipzig applicators

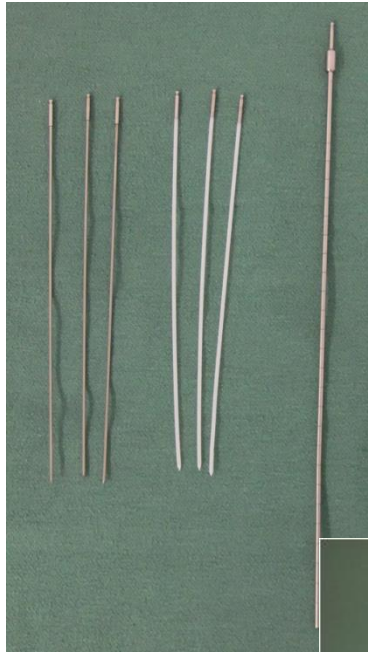


Gynecological applicators

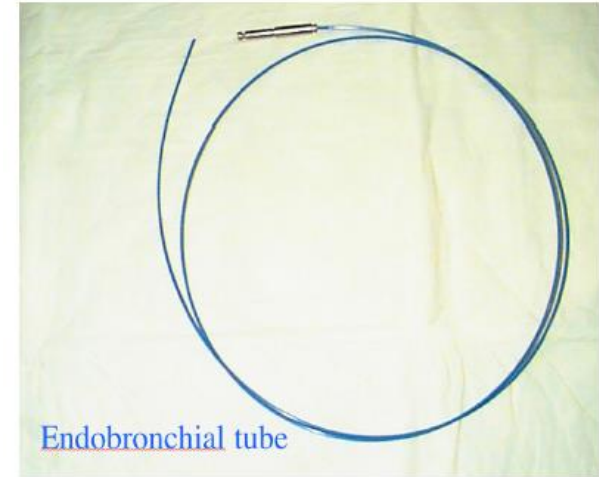
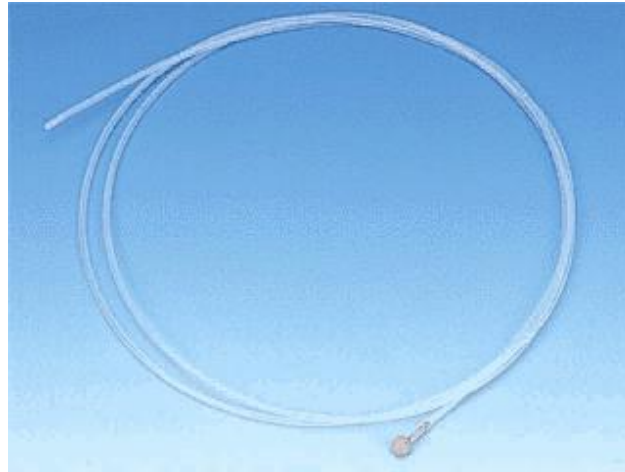


Brachytherapy – Applicators

Needles



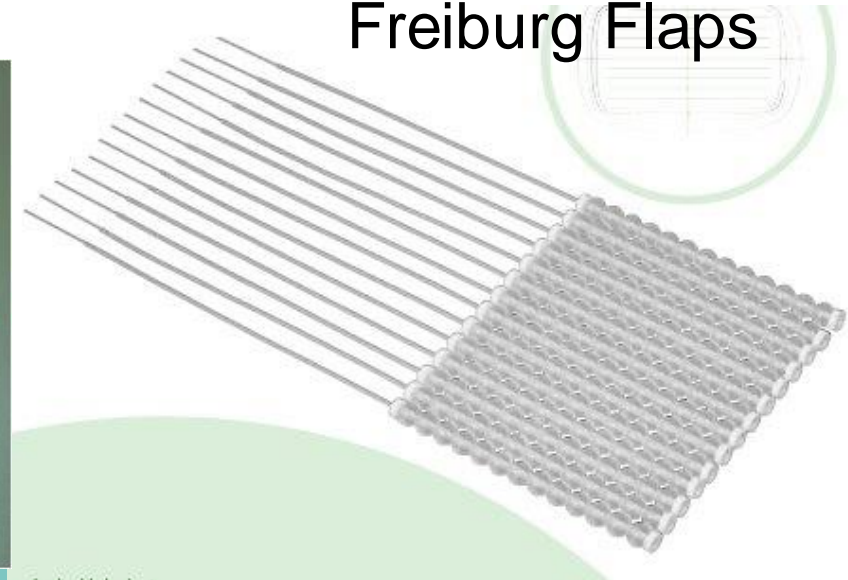
Catheters



Breast applicators



Freiburg Flaps



Dosimetry Systems

Dosimetry Systems

Dosimetric systems are set of rules, specific to a radioisotope and its spatial distribution in the applicator to deliver a defined dose to a designated region

Within any system, specification of treatment in terms of dose, timing, and administration is necessary so as to implement prescription in a reproducible manner

Historical Dosimetry Systems – Uterine Cervix

Stockholm

- Radium
 - 2-3 applications within about a month
 - Intravaginal boxes made up of silver or lead
 - 60-80 mg of Radium
 - Intrauterine tube made up of rubber
 - 30-90 mg of Radium
 - 20-30 hours per session
- ➔ Total dose 6500 - 7100 mg hrs

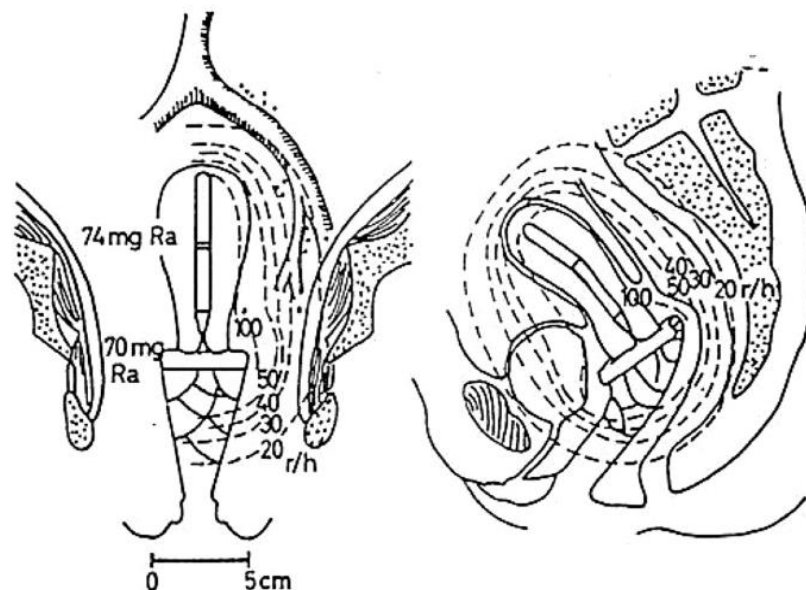


Figure 3.1. The Stockholm system. Typical treatment of a cervix carcinoma with a radium application (uterus normal in size and shape). The total amount of radium is 74 mg + 70 mg = 144 mg of radium. The exposure rates (roentgen per hour) are shown in the frontal and sagittal plane. An amount of 144 mg of radium (1 mm Pt filtration) delivers a total reference air-kerma rate of $964 \mu\text{Gy h}^{-1}$. After 3 applications of 27 h each, the TRAK would be 78 mGy and a source-mass \times duration of 11.664 mg h (Walstam, 1954) (from ICRU 1985). The relationship between exposure, absorbed dose, and air kerma is described in Section 11.

Historical Dosimetry Systems – Uterine Cervix

Paris

- Radium
- Single application
- 2 cork colpostats in the vagina
 - 13.3 mg of Radium each
- Intrauterine tube made of thin rubber with 3 radioactive sources
 - 33.3 mg
 - source strengths distribution: 1:1:0.5
- 120 hours
- ➔ Total dose of 7000-8000 mg hrs

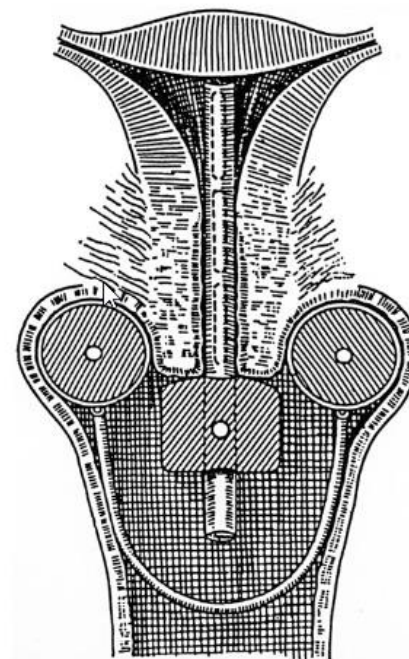


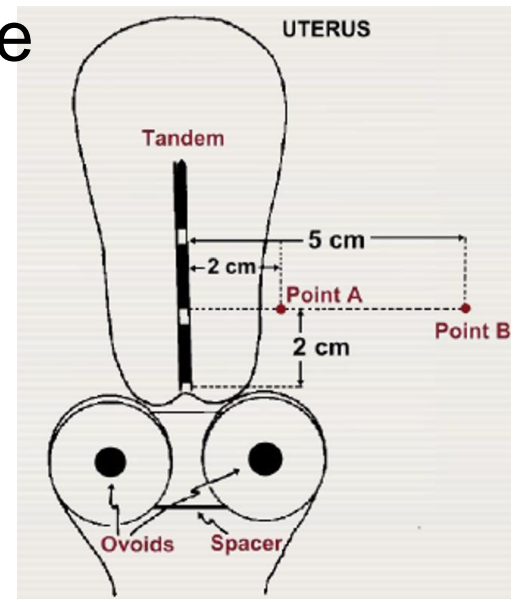
Figure 3.2. Schematic representation of the “historical” Paris method. Assuming three intra-uterine sources of 10 mg radium each and 3 intra-vaginal sources of 10 mg each, the total activity of 60 mg (1 mm Pt filtration) would deliver a TRAK rate of $402 \mu\text{Gy h}^{-1}$. After an application of 6 days (144 h), the TRAK would be 57.9 mGy and a source-mass \times duration of 8.640 mg h (ICRU, 1985; Pierquin 1964).

ICRU 89 (2013): Prescribing, Recording, and Reporting Brachytherapy for Cancer of the Cervix

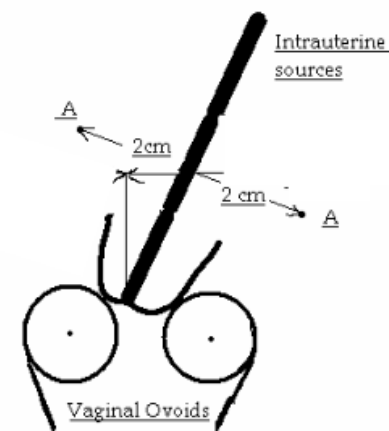
Historical Dosimetry Systems – Uterine Cervix

Manchester system is characterized by the dose to following points:

- Point A: Dose prescription point. Define the duration of the irradiation
- Point B: Describe the dose fall off outside the target. It gives the dose in the vicinity of the pelvic wall near the obturator nodes.
- Bladder point
- Rectum point



If the tandem displaces the central canal, point A moves with the canal, but point B remains fixed at 5 cm from the midline.



Historical Dosimetry Systems – Uterine Cervix

Manchester

- Radium
- Standard applicators
 - Pair of ovoids: 40% of the dose to point A
 - Intrauterine tube: 60% of the dose to point A
- Total dose at point A: 8000 mg hrs
- 2 sessions
- ➔ Total irradiation time: 144 hours

Historical Dosimetry Systems

Because of the inherent radiation safety risk, ^{226}Ra has been progressively abandoned and is forbidden in some countries and by several authorities.

It has been replaced by artificial radionuclides, such as ^{60}Co , ^{137}Cs , and ^{192}Ir .

The lower energy of the gamma emissions of ^{137}Cs and ^{192}Ir also simplifies the practical problems of room shielding and reduces the exposure to staff.

Historical Dosimetry Systems

The replacement of radium by ^{137}Cs , ^{192}Ir and ^{60}Co followed one of two options:

- The new sources (^{137}Cs or ^{60}Co) were similar in size and shape and had an output similar to radium sources. The same technique of application could then be used, and the clinical experience gained with radium remained fully relevant.
- Using ^{192}Ir takes advantage of improved technology in the preparation of the sources:
 - increased specific activities
 - miniaturized sources

Dosimetry Systems – ICRU 38 (GYN)

For a reliable comparison of the different methods and their results ICRU 38 recommends a common terminology for prescribing recording and reporting of intracavitary brachytherapy applications.

ICRU 38 recommend a system of dose specification that relates to a dose distribution to the target volume instead of the dose to a point.

➔ Instead of prescribing the dose to a point, the dimensions of the volume included in the **reference dose level of 60 Gy**.

The 60 Gy reference volume may be equal to or different from the treated volume. The treated volume should encompass at least the clinical target volume (CTV).

The reference dose level of 60 Gy is for LDR. For HDR a reference dose lower than 60 Gy is recommended

When intracavitary therapy is combined with external beam therapy, the dose level to define the reference volume is the difference between 60 Gy and the dose delivered by external beam therapy.

Dosimetry Systems – ICRU 38 (GYN)

- Description of the technique
 - Applicator type
 - Source type, strength and arrangement
 - Loading technique
- Total reference air kerma
- Description of the reference volume
 - Dimensions: Height (d_h), width (d_w) and thickness (d_t) of the pear shaped isodose
- Absorbed dose at reference points
 - Point A
 - Point B
 - Bladder point
 - Rectum point
 - Lymphatic Trapezoid of Fletcher
 - Pelvic Wall points
- Time dose pattern
 - Dose rate and duration

Dosimetry Systems – 2D → 3D

ICRU 38 mainly provide recommendations for applications planned on the basis of radiographs.

→ ICRU 89 (GYN)

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•	4 - BRACHYTHERAPY IMAGING FOR TREATMENT PLANNING
•	5 - TUMOR AND TARGET VOLUMES AND ADAPTIVE RADIOTHERAPY
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•	7 - RADIOBIOLOGICAL CONSIDERATIONS
•	8 - DOSE AND VOLUME PARAMETERS FOR PRESCRIBING, RECORDING, AND REPORTING OF BRACHYTHERAPY ALONE AND COMBINED WITH EXTERNAL BEAM RADIOTHERAPY
•	9 - 3D VOLUMETRIC DOSE ASSESSMENT
•	10 - RADIOGRAPHIC DOSE ASSESSMENT
•	11 - SOURCES AND DOSE CALCULATION
•	12 - TREATMENT PLANNING
•	13 - SUMMARY OF THE RECOMMENDATIONS
•	APPENDIX – EXAMPLES, SPREADSHEETS, DRAWINGS

BrachyNext – Working Together to Shape the Future of Brachytherapy

ICRU 89 (GYN)

Use imaging (US, CT, MRI, PET) to:

- conform the dose to the target
- Effectively spares OARs

MRI:

- Superior soft tissue resolution
- HRCTV smaller than on CT
- Greater conformality will lead to decrease dose to OARs
 - Possibly more critical for large lesions
- First fraction or every fraction
 - Beware of significant tumor response
 - $T_{1/2}$ for tumor response 20-21 days (CT, MR, clinical exam)

Historical Dosimetry Systems – Interstitial

The most commonly used systems in interstitial Brachytherapy were:

- Patterson-Parker (Manchester) system
- Paris system
- Quimby (Memorial) system

Historical Dosimetry Systems – Intertitial

Manchester (Patterson-Parker) System

The radioactive sources are distributed non-uniformly over the area or the volume of the implant to give a uniform dose distribution. The aim of the system is to deliver a uniform dose of $\pm 10\%$ in the implanted area or volume.

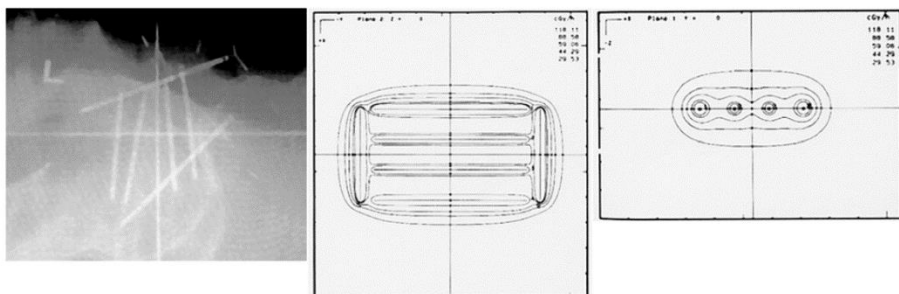


Fig 6.2: Manchester System for application of radioactive sources with different loading. Fig A shows the localisation film. Fig B and C give the distribution of dose rate for a single-plane implant with iridium wires of unequal linear activity in order to ensure dose uniformity throughout the implanted region. Wires 1, 4, 5 and 6 (peripheral) contain a linear activity of 60 MBq per cm; wires 2 and 3 contain a linear activity of 37MBq per cm. Wires 1, 2, 3 and 4 are 6 cm long; wires 5 and 6 are 3.5 cm long. Fig B gives the dose rates in the plane containing the wires, Fig C in a perpendicular plane. (From Wambersie and Battermann [115])

Patterson-Parker tables

Single plane

This source arrangement treats a 1 cm thick slab of tissue. The prescribed dose is on a parallel plane 0.5cm away from the source plane

Double plane

It is used to treat slabs with thickness between 1cm and 2.5cm. The required total source strength is equally divided between the two planes

Historical Dosimetry Systems – Intertitial

Paris System

Paris system was developed for linear sources of iridium wires. It is used for single or double plane implants. In this system, sources are distributed uniformly for a planar implant but follow a particular pattern for volume implants:

- Sources must be linear and their placement must be parallel
- Centres of all sources must be located in the same (central) plane
- Linear source strength (activity) must be uniform and identical for all sources in the implant
- Adjacent sources must be equidistant to each other

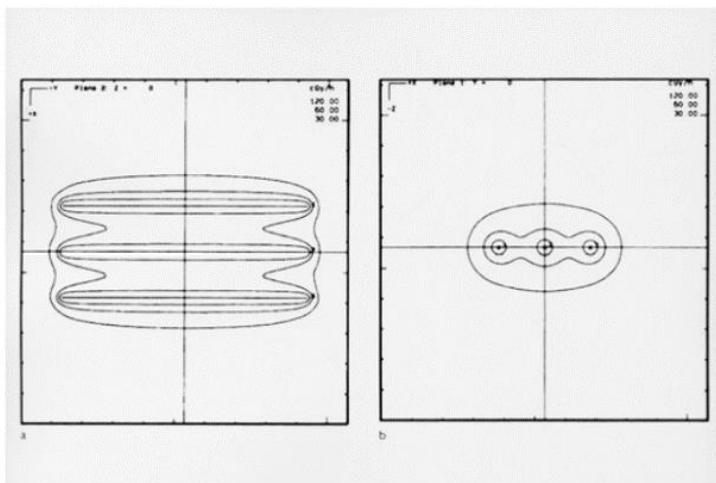
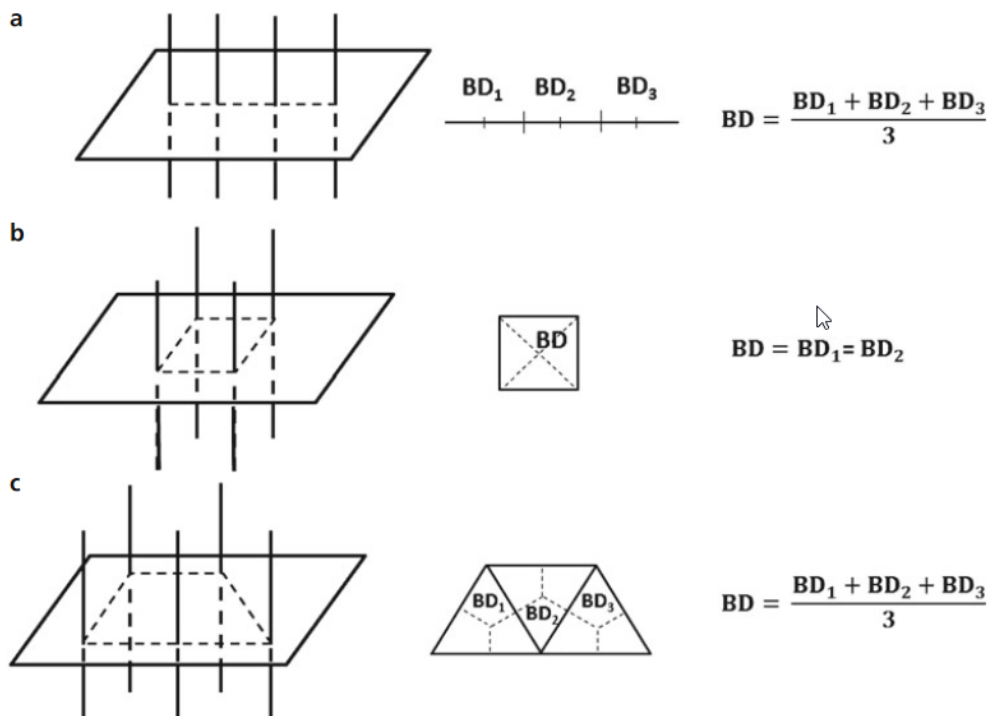


Fig 6.3 : Iridium-192 wire implant according to the Paris system (single-plane implant). The wires are of equal linear activity, parallel, and arranged in such a way that their centres are in the same plane perpendicular to the direction of the wires (i.e. the central plane, see Fig 6.4). (From Wambersie and Battermann [115])

Historical Dosimetry Systems – Intertitial

Paris System

The Paris system recommends as a guideline for the reference dose rate (i.e. the dose rate with which the irradiation time is determined) 85% of the basal dose rate.



From: Dutreix A, Marinello G, Wambersie A (1982) Dosimetrie en Curietherapie

Historical Dosimetry Systems – Intertitial

Quimby (Memorial) System

The radioactive sources are distributed uniformly over a plane or a volume of tissue. The dose near the center of the implant is much greater than at the edge of the implant. Thus in Quimby system, a uniform distribution of radioactive sources is used to give a non-uniform dose distribution.

Dosimetry Systems – ICRU 58

The aim of the ICRU 58 was to develop a common language that was based on the presently existing concepts.

The ICRU 58 was written to generate a guideline about the dose specification and reporting for interstitial brachytherapy.

Dosimetry Systems – ICRU 58 reporting

- Description of the clinical target volume
- Sources
- Source strength
 - Reference air kerma rate in air [cGy/h @ 1m]
- Technique and implant time
- Prescription dose
- Description of the dose distribution
 - Total reference air kerma
 - Mean central dose
 - Minimum dose
 - High dose regions
 - Dose uniformity indices
- Dose volume histograms (DVH)

Dosimetry Systems/Planning – Reconstruction

Source/implant/applicator localization:

2D

- Projections

3D

- Computerized tomography (CT) scanning
- Ultrasound scanning (US)
- Magnetic resonance imaging (MRI)



Dosimetry Systems/Planning – Afterloading

The target volume is covered with a sufficient radiation dose by introducing a sufficient number of spatially suitably distributed source positions into the tissue. The planning and preparation of the irradiation takes place in 3 steps:

- 1 Determination of a suitable application geometry to enable a suitable distribution of the source positions and insertion of the applicators.
- 2 Reconstruction of the application geometry based on CT, MR, radiography or ultrasound images.
- 3 Optimization: Determination of the source positions and dwell times. Steps 2 and 3 are usually carried out with computer-aided treatment planning systems.

Dosimetry Systems/Planning – Optimization

Geometrical optimization

The goal of the geometric optimization is to reduce the inhomogeneity in a given distribution of the source positions by adjusting the dwell times at given source strengths :

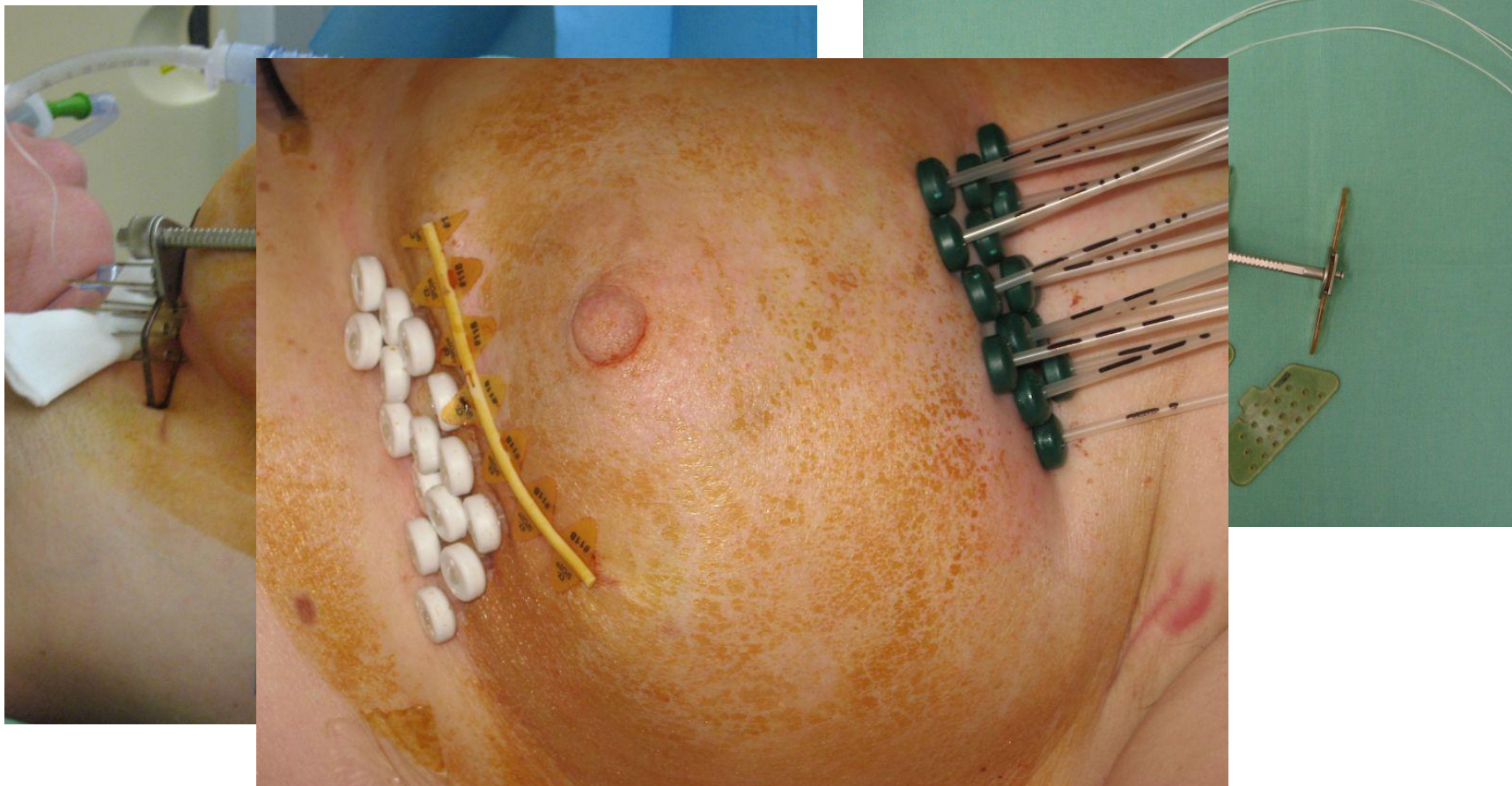
- Distance Optimization
- Volume Optimization
- ...

Anatomy based optimization

The aim of the anatomy based optimization is to meet the target requirements set by the planner on given structures of the anatomy by modulating the dwell times of the source positions or by specifying seed locations at given source strengths:

- HIPO
- IPSA
- Manual, ...

Brachytherapy – Mamma Ca

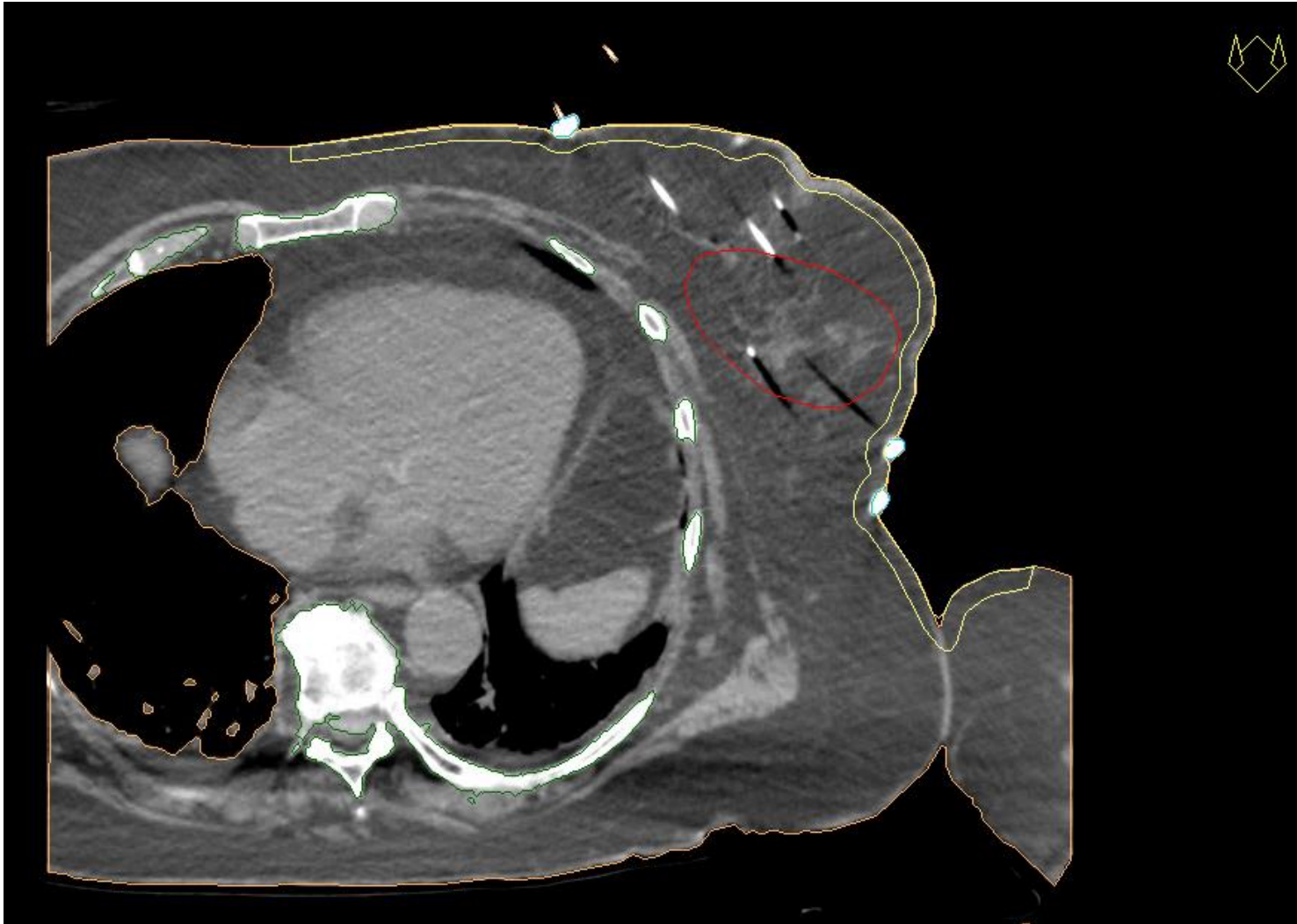


Mamma Ca: Interstitial brachytherapy, partial breast radiation

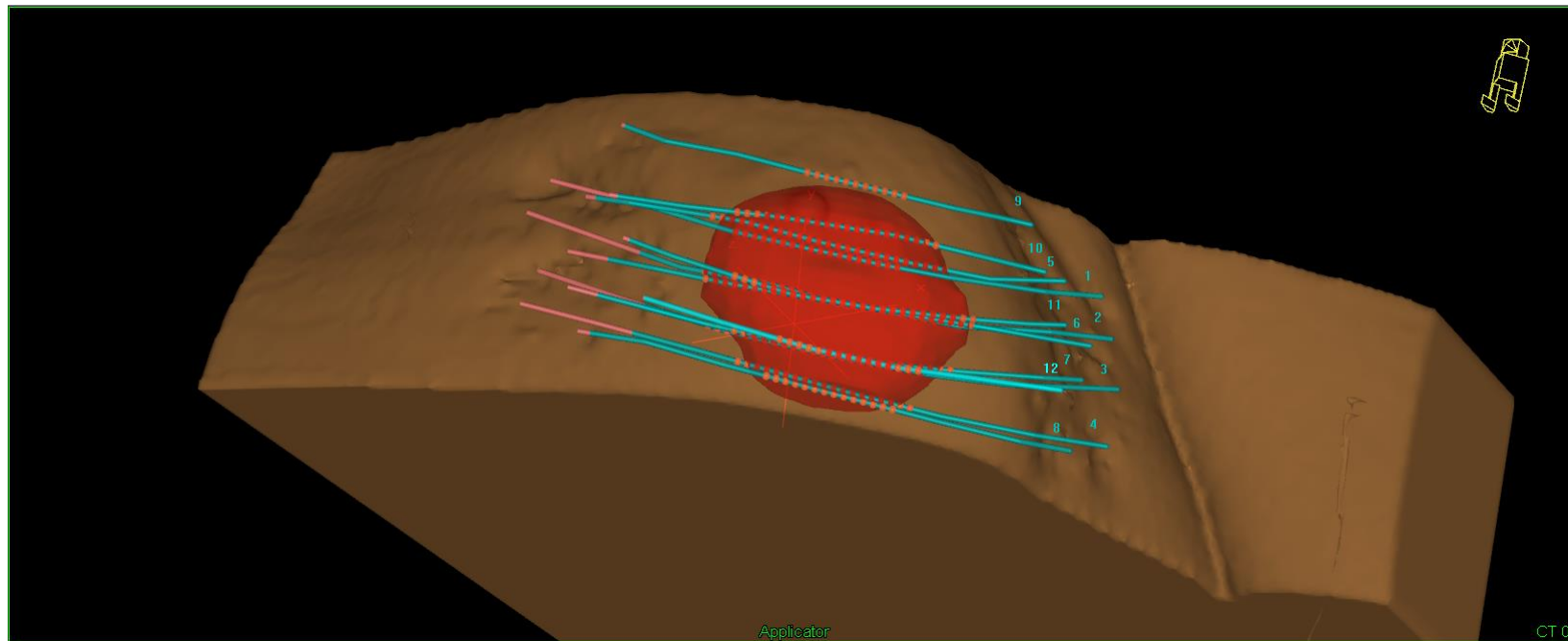
Planning – CT acquisition



Planning – Contouring



HDR Planning - Activation

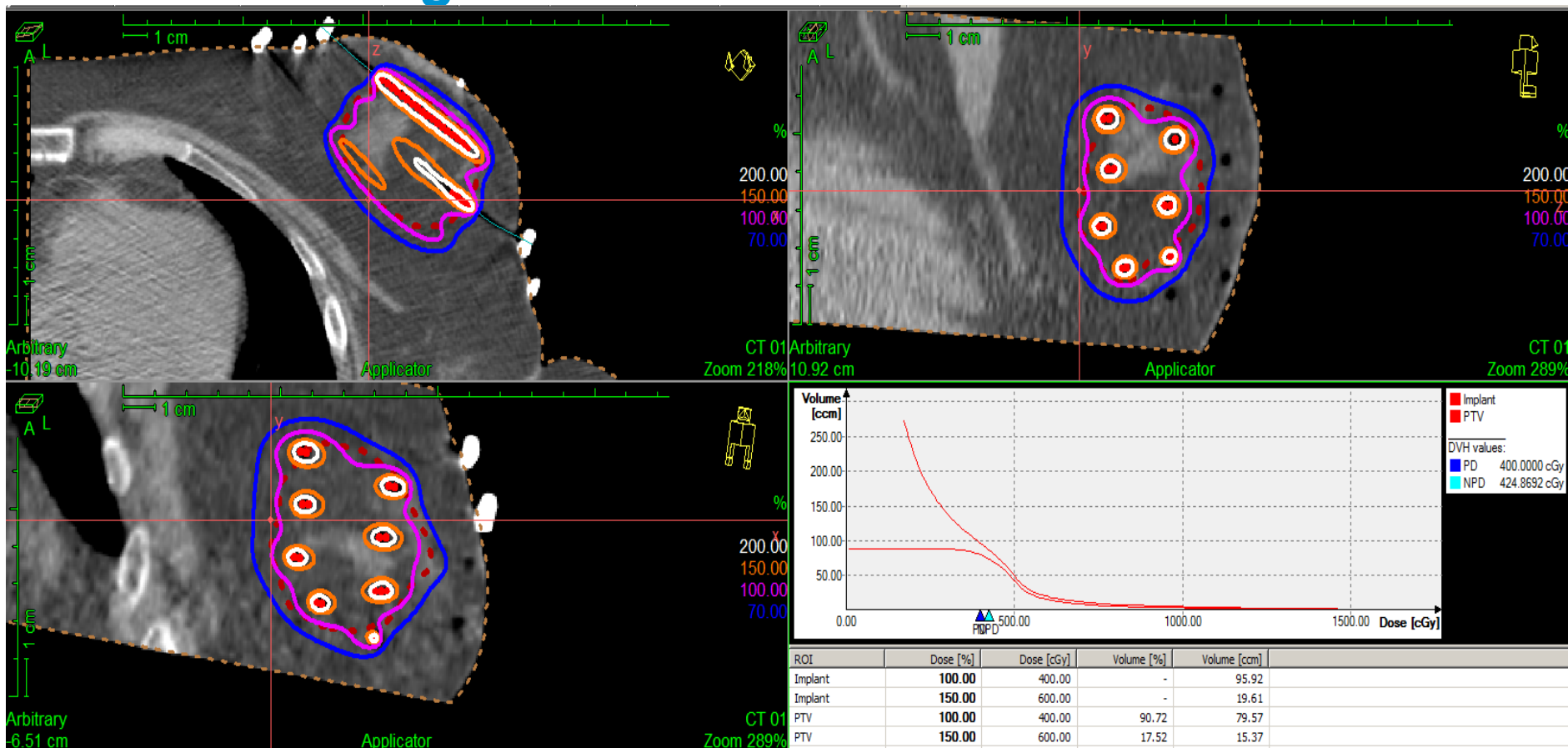


CT 01

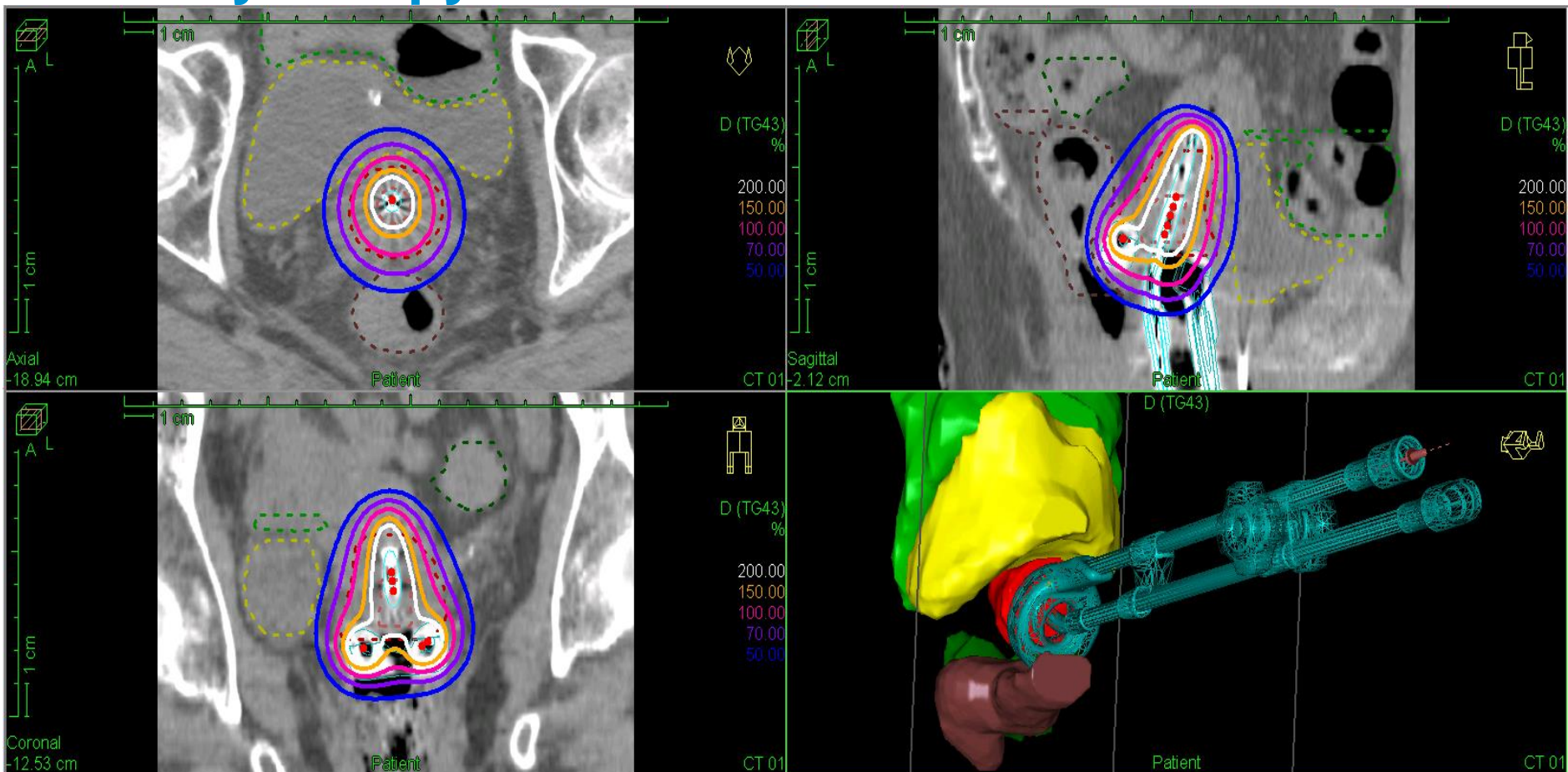
Cath.#	Ch.#	Name	Lock (HIPO)	Indexer [cm]	Offset [cm]	0.0	1.0	2.0	3.0	4.0	5.0	6.0	7.0	8.0	9.0	10.0	11.0
1	1	(Manual)		128.30	-0.55												
2	2	(Manual)		128.40	-0.55												
3	3	(Manual)		128.00	-0.55												
4	4	(Manual)		126.40	-0.55												
5	5	(Manual)		127.80	-0.55												
6	6	(Manual)		127.90	-0.55												
7	7	(Manual)		127.50	-0.55												
8	8	(Manual)		126.30	-0.55												
9	9	(Manual)		126.70	-0.55												
10	10	(Manual)		125.70	-0.55												
11	11	(Manual)		127.20	-0.55												
12	12	(Manual)		125.60	-0.55												

The table shows the configuration for 12 applicators. The 'Offset' column is set to -0.55 cm for all. The 'Indexer' column shows the distance from the applicator tip to the source. The grid to the right of the table shows the source positions for each applicator, with red dots indicating the source locations. The grid is 12 columns wide, corresponding to the 12 applicators. The source positions are distributed across the grid, with some applicators having multiple source positions. The grid also shows some greyed-out areas, likely representing non-activated sources.

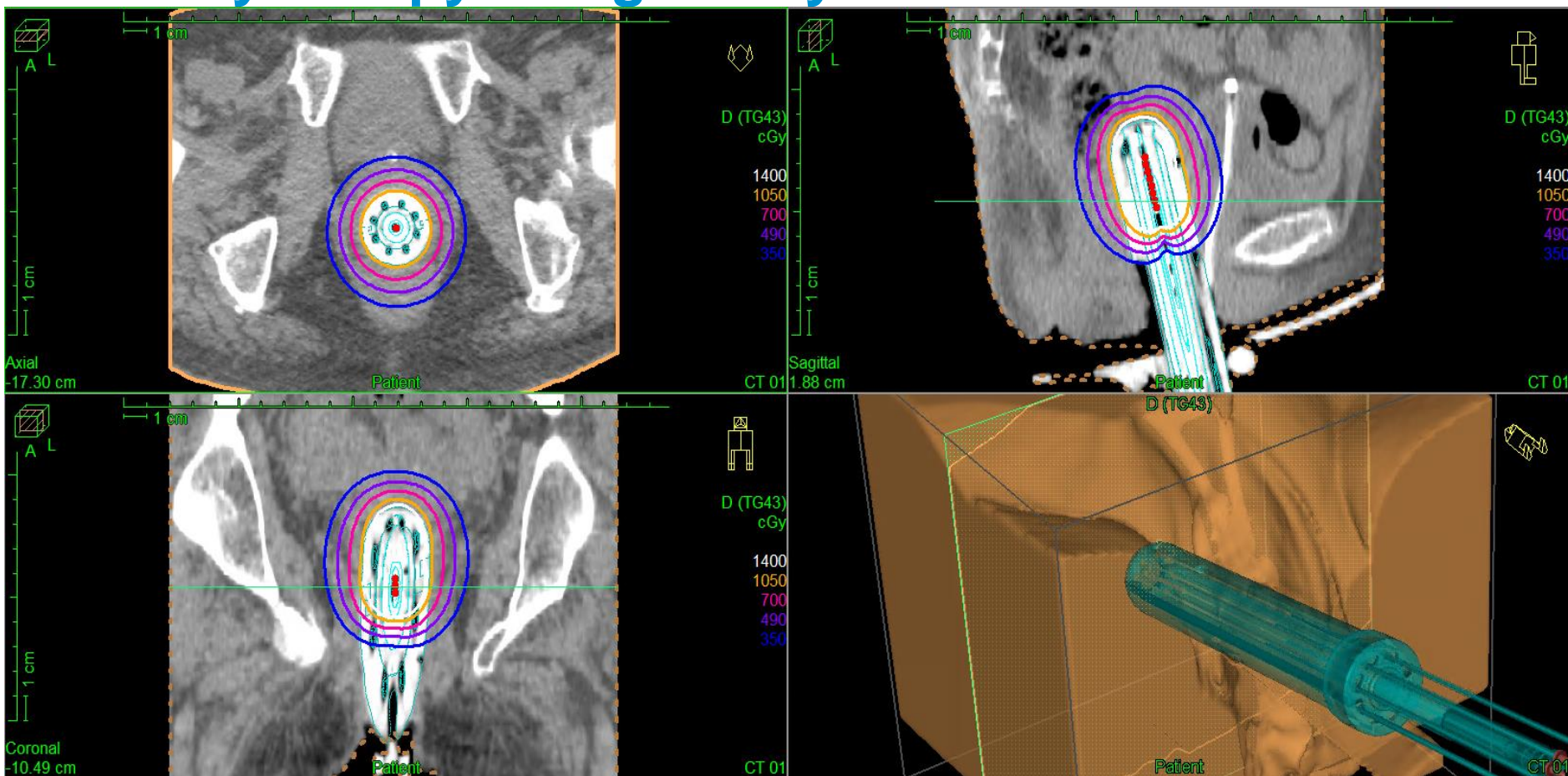
HDR Planning



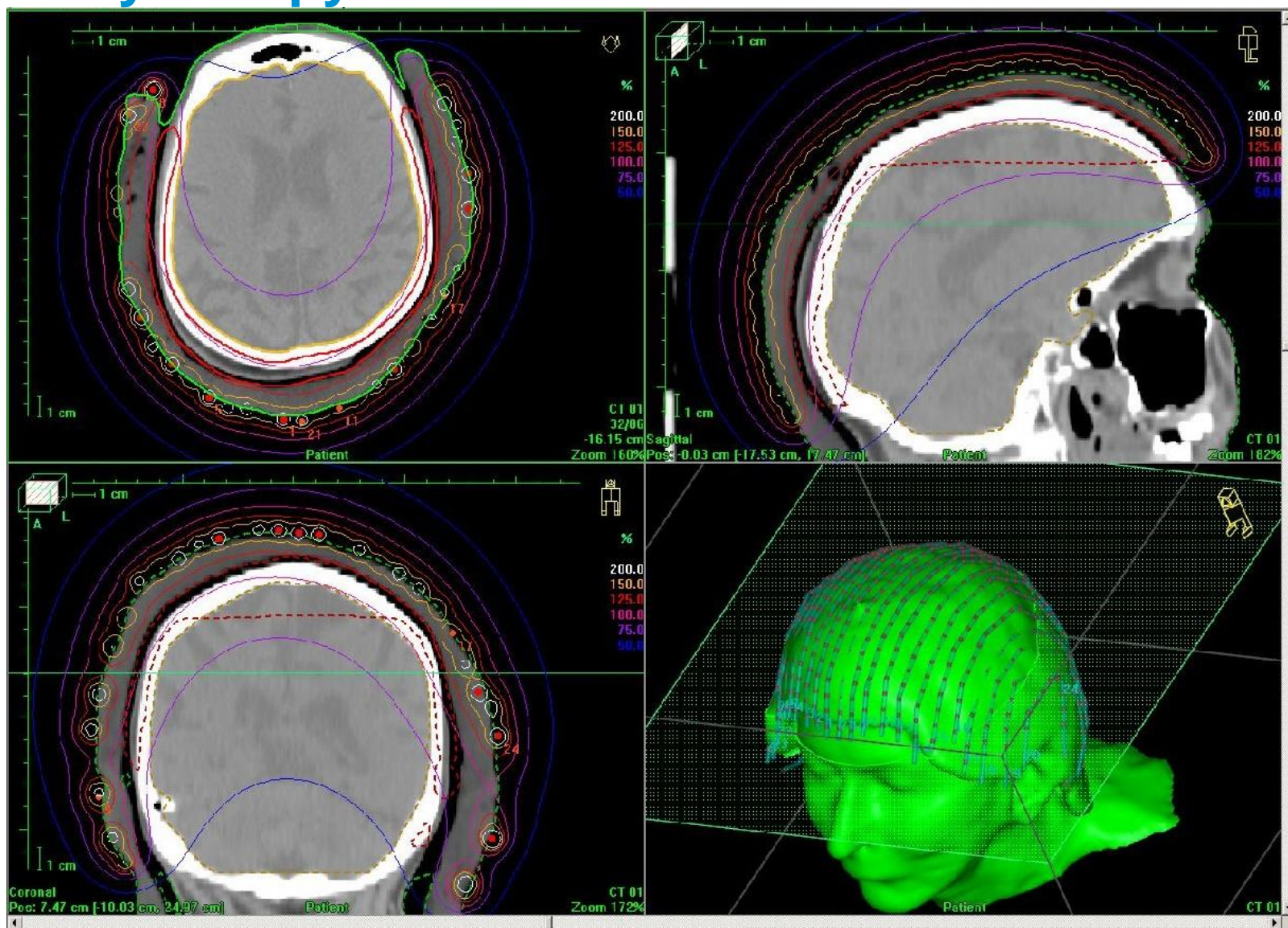
Brachytherapy – Intrauterine



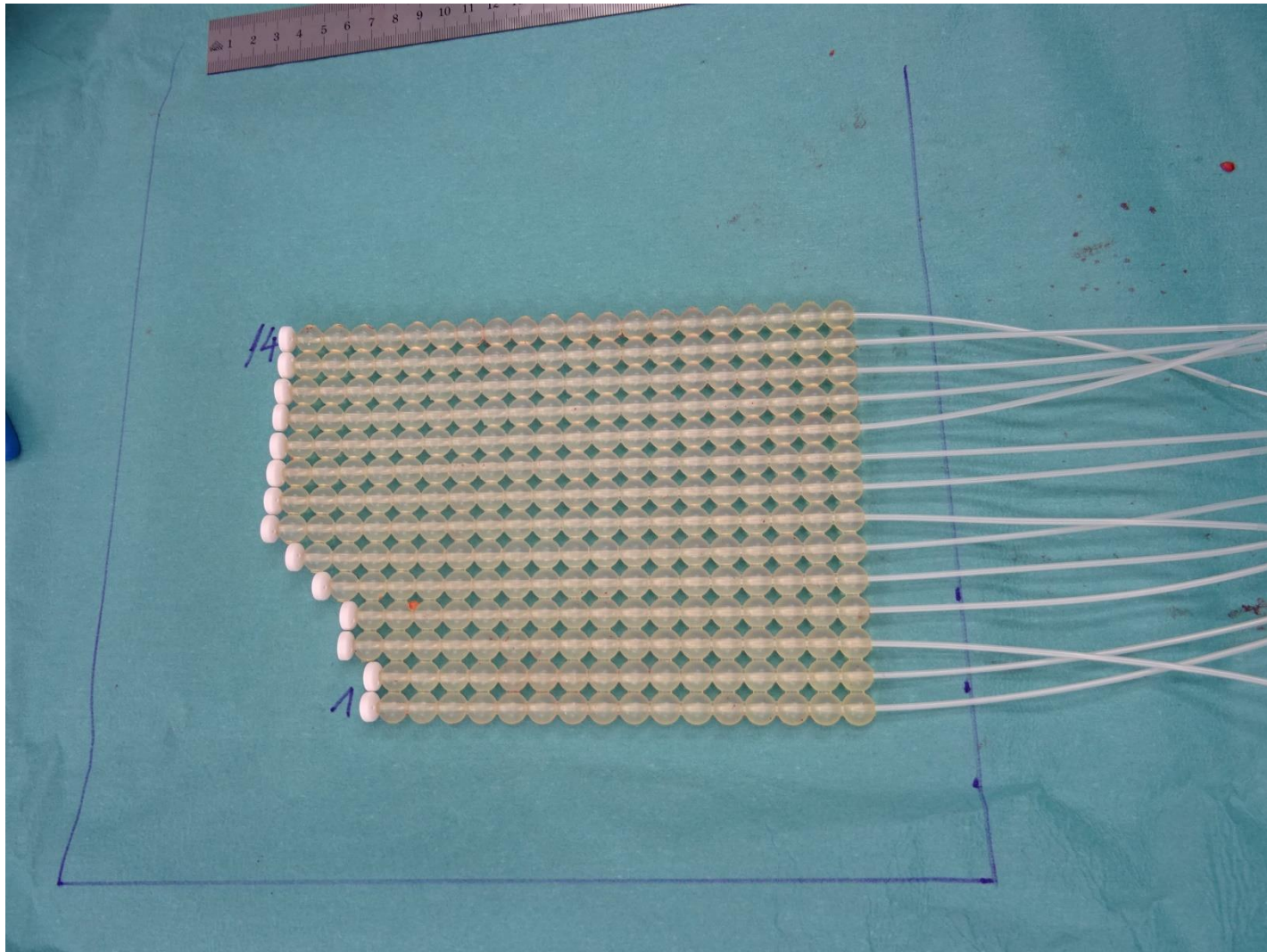
Brachytherapy – Vaginal Cylinder



Brachytherapy – Skull



Brachytherapy – Intraoperative Brachytherapy



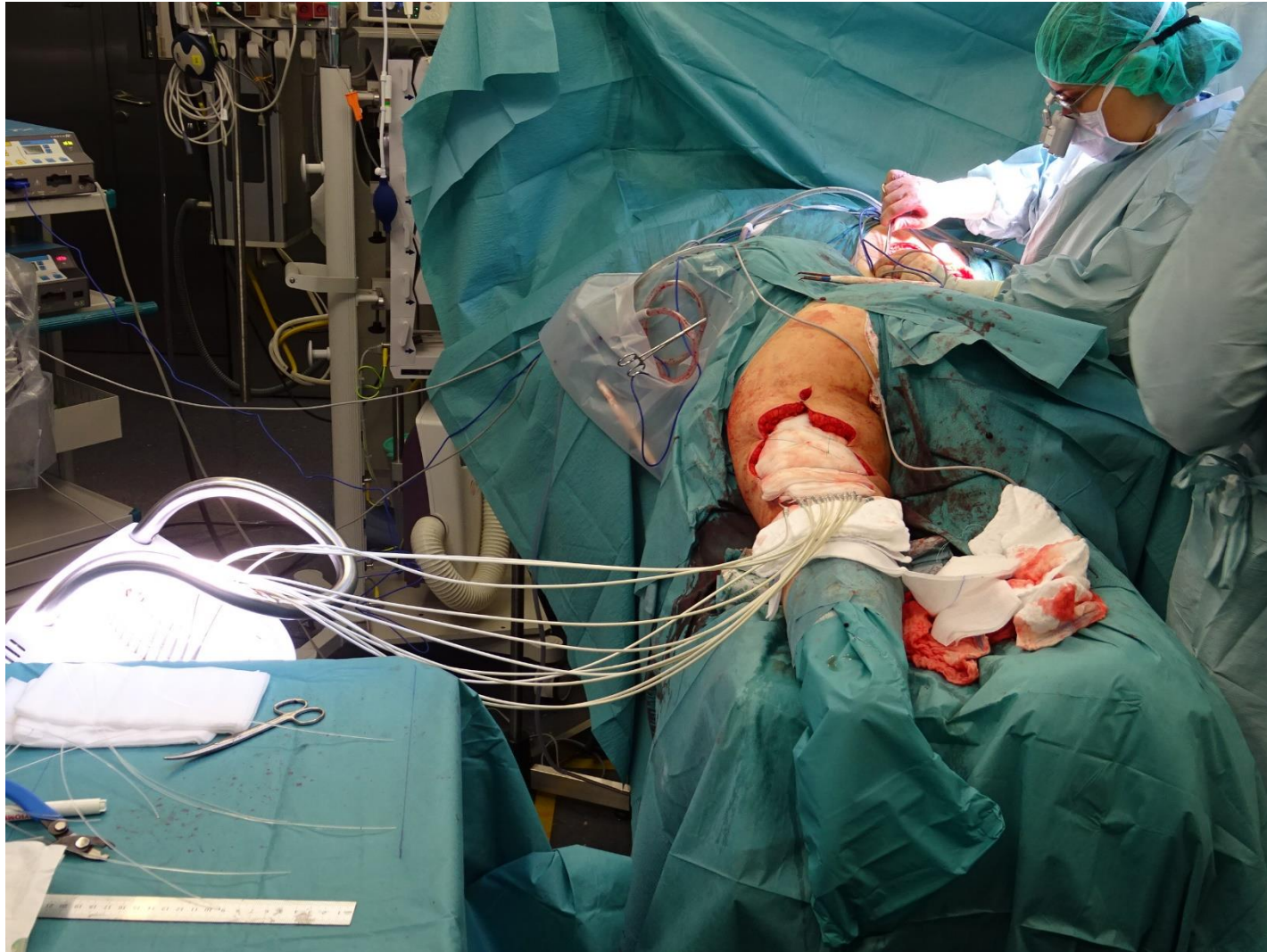
Brachytherapy – Intraoperative Brachytherapy



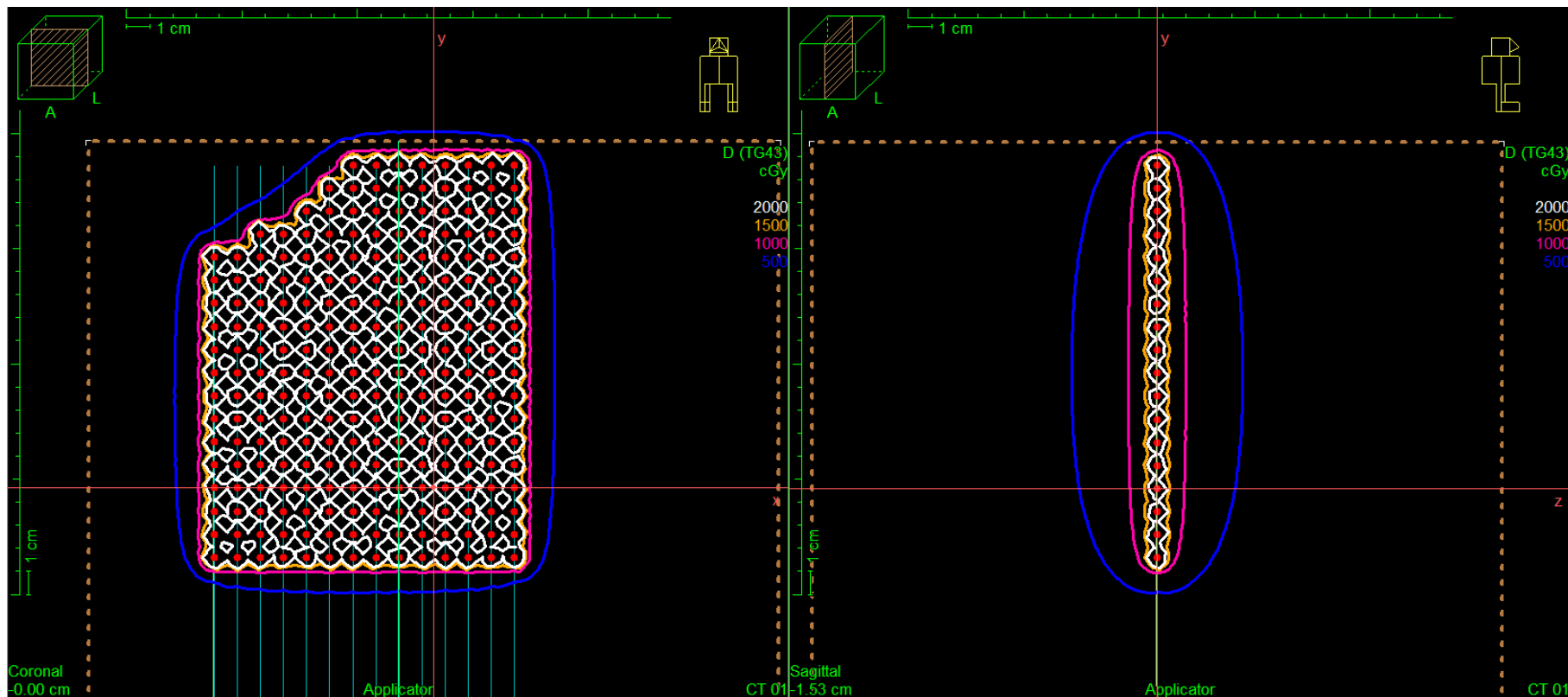
Brachytherapy – Intraoperative Brachytherapy



Brachytherapy – Intraoperative Brachytherapy



Brachytherapy – Intraoperative Brachytherapy



Brachytherapy – Intraoperative Brachytherapy

At the Inselspital IORTs are carried out for:

- Tumors of the gastrointestinal tract
- Sarcomas
- Gynecological tumors
- Recurrent tumors

Prescription dose: 10 Gy @ 5mm tissue depth

Dose Calculation

- Point Source approximation
- TG43
- ACE (Collapsed Cone)
- Full Monte Carlo
- Acuros (Boltzmann Solver)

Dose Calculation

- Point Source approximation
- AAPM-TG43
- ACE (Collapsed Cone)
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Today – Point Source

$$\dot{D}_{water}(r) = \dot{K}_{air}(r_{ref})_{air} \cdot \left[\frac{\bar{\mu}_{tr}}{\rho} \right]_{Air}^{Water} \cdot (1 - g) \cdot T(r) \cdot \left(\frac{r_{ref}^2}{r^2} \right)$$

where:

$\left[\frac{\bar{\mu}_{en}}{\rho} \right]_{Air}^{Water}$: ratio of the photon mass energy absorption coefficients (water to air).

T(r) : Tissue attenuation and scatter function :

$$T(d) = \dot{K}_{air}(d)_{water} / \dot{K}_{air}(d)_{air} = a_0 + a_1 \cdot r + a_2 \cdot r^2 + a_3 \cdot r^3$$

Meisberger L.L., Keller R.J. and Shalek R.J.:
 “The effective attenuation of the gamma rays of gold-198, Iridium-192, Caesium-137, Radium-226 and Cobalt-60”. Radiology 90, 953-957, 1968.

Nuklid	Material	Polynomkoeffizienten				Autoren
		a ₀	a ₁	a ₃	a ₄	
192-Ir	H ₂ O	1.0128	5.019E-3	-1.178E-3	-2.008E-5	Meisberger
	H ₂ O	1.0380	1.862E-3	-1.300E-3	1.865E-5	Krieger 3
	Polystyrol*	0.9970	0.840E-2	1.136E-1	-2.140E-4	Kneschaurek
137-Cs	H ₂ O	1.0091	-9.015E-3	-3.459E-4	-2.817E-5	Meisberger
60-Co	H ₂ O	0.9942	-5.318E-3	-2.610E-3	1.327E-4	Meisberger

Polynomkoeffizienten für die radialen Tiefendosiskurven von Afterloadingstrahlern im Phantom (nach Gl. 6.16, *: zusätzliches Exponentialglied e^{-μr} mit μ = 0.113 cm⁻¹).

From: Krieger, «Strahlenphysik, Dosimetrie und Strahlenschutz», Band 2, B.G.Teubner Stuttgart 1997

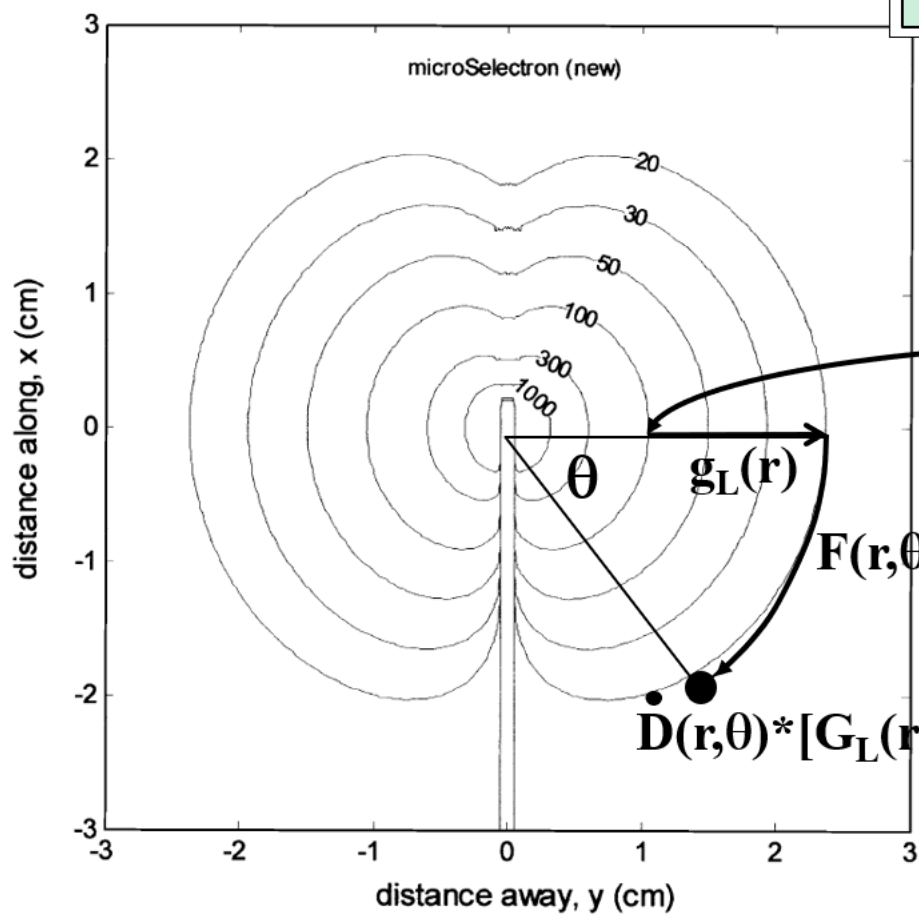
Dose Calculation

- Point Source approximation
- **AAPM-TG43**
- ACE (Collapsed Cone)
- Full Monte Carlo
- Acuros (Boltzmann Solver)

TG43 – Formalism

$$\dot{D}(r, \theta) = S_k \cdot \Lambda \cdot \frac{G_L(r, \theta)}{G_L(r_0, \theta_0)} \cdot g_L(r) \cdot F(r, \theta)$$

Calculated with MC



$$\dot{D}(r, \theta) * [G_L(r_0, \theta_0) / G_L(r, \theta)]$$

Λ

S_k

TG43 – Formalism

Kerma Strength: $S_k = \dot{K}_\delta(b) \cdot b^2$ Measured for each source individually.

Dose Rate Constant: $\Lambda = \frac{\dot{D}(r_o, \theta_o)}{S_k}$ Measured or calculated with Monte Carlo for each source model.

Radial Dose Function: $g_L(r) = \frac{\dot{D}(r, \theta_o)}{\dot{D}(r_o, \theta_o)} \cdot \frac{G_L(r_o, \theta_o)}{G_L(r, \theta_o)}$

Anisotropy Function: $F(r, \theta) = \frac{\dot{D}(r, \theta)}{\dot{D}(r, \theta_o)} \cdot \frac{G_L(r, \theta_o)}{G_L(r, \theta)}$

Calculated with Monte Carlo for each source model.

with:

$$r_o = 1\text{cm}$$

$$\Theta_o = 90^\circ$$

TG43 – ...some numbers for our Ir-192 Source

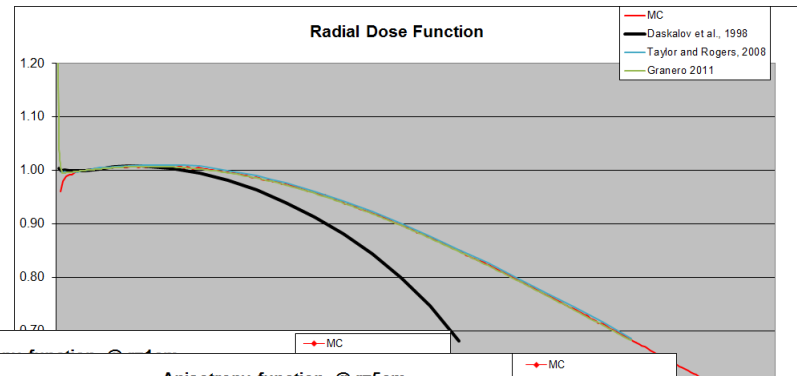
Kerma Strength: $S_k = 30'000 \frac{\mu\text{Gy} \cdot \text{m}^2}{\text{h}} \left(= 3 \frac{\text{cGy} \cdot \text{m}^2}{\text{h}} \right)$

$1U = 1 \frac{\mu\text{Gy} \cdot \text{m}^2}{\text{h}}$

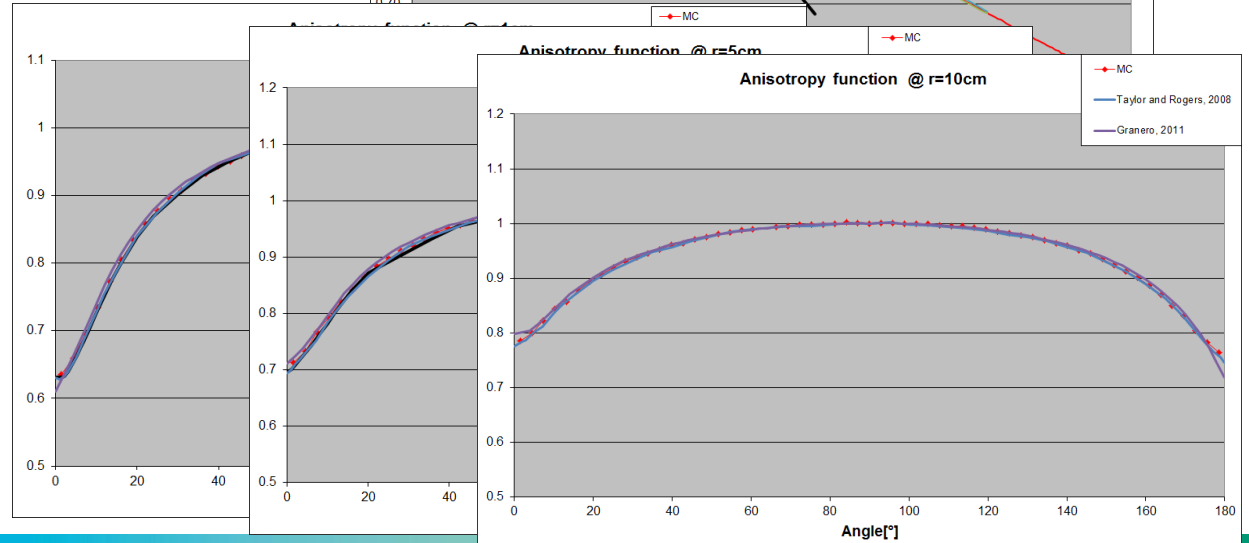
(Typical value after source replacement)

Dose Rate Constant: $\Lambda = 1.108 \frac{\text{cGy}}{\text{h} \cdot \text{U}}$

Radial Dose Function:

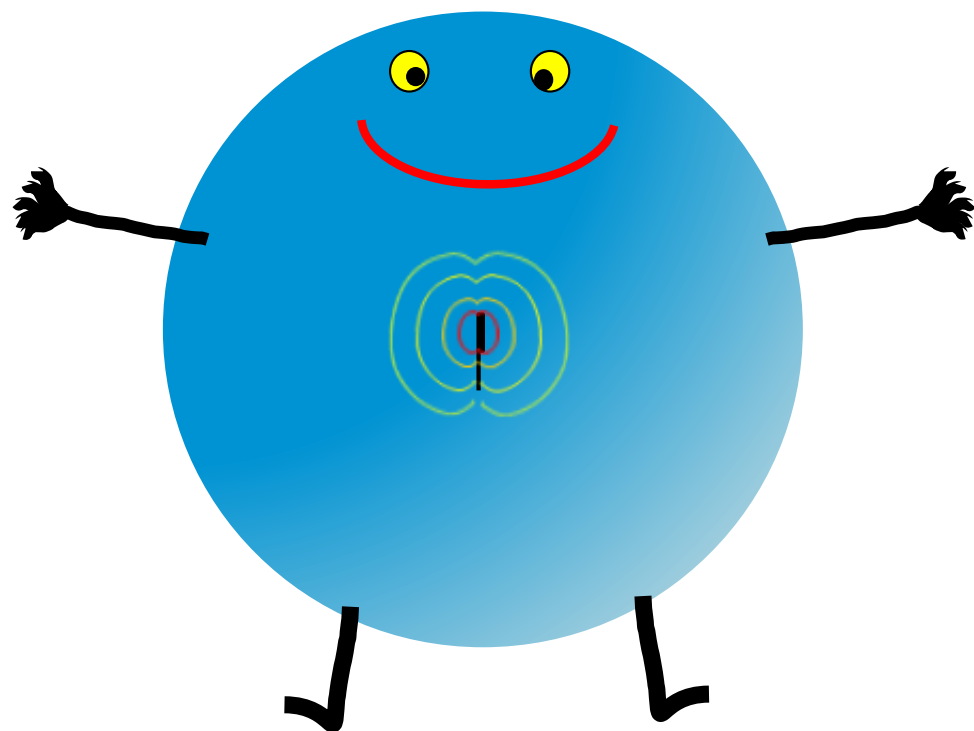


Anisotropy Function:



TG43 – Basic assumptions (...and limitations)

- Dose calculation in water
 - no material heterogeneities within the body
 - no applicators
 - no shieldings
 - no source interplay
- Infinite patient
 - no phantom size effect
- Azimuthal symmetry



TG43 – Shieldings & Applicators

Fletcher-Williamson colpostat

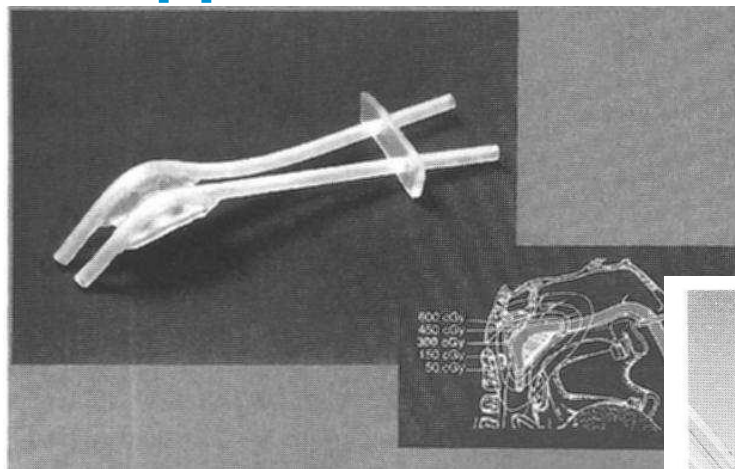
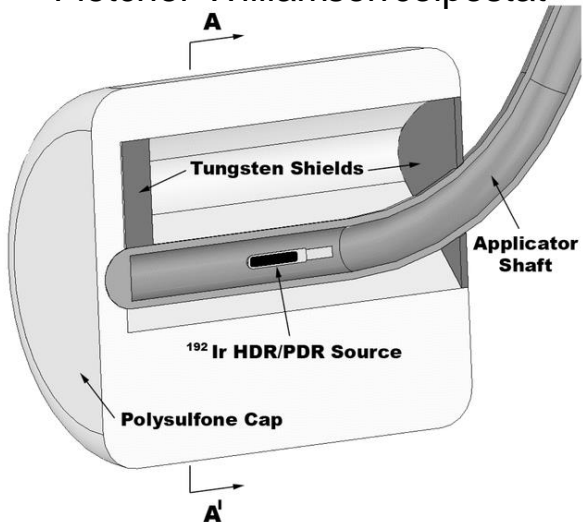


Figure 5. HDR tandem and cylinder applicator.

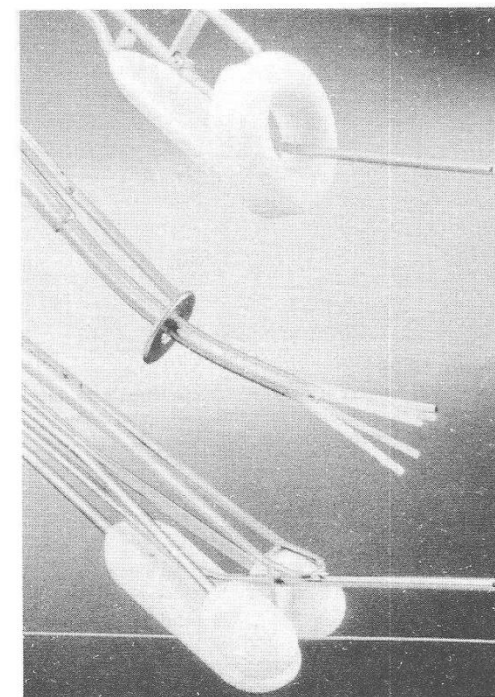
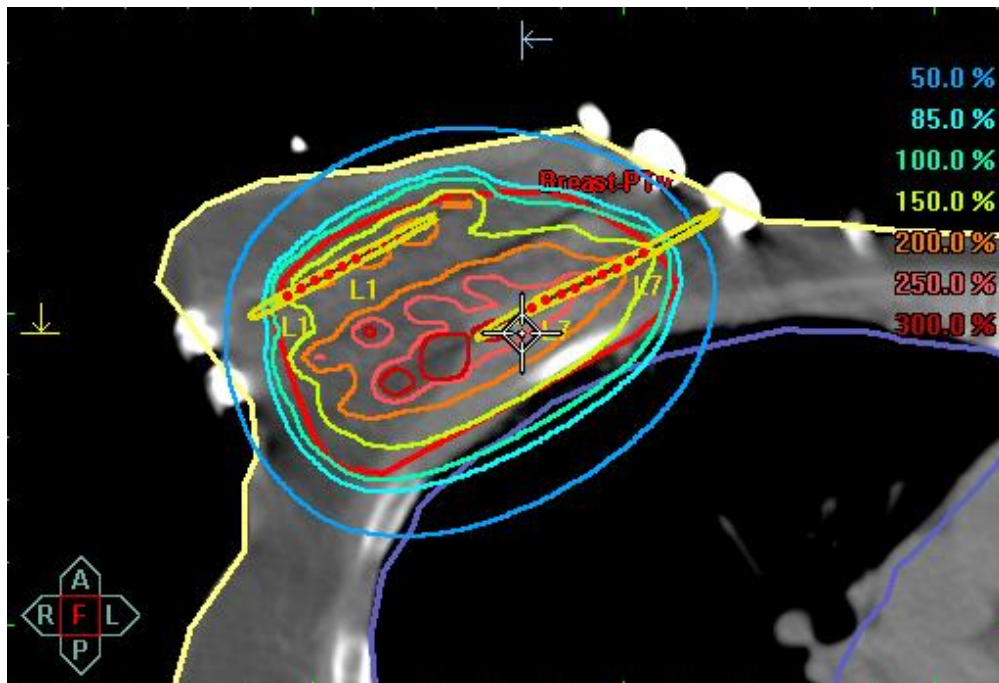


Figure 18.27

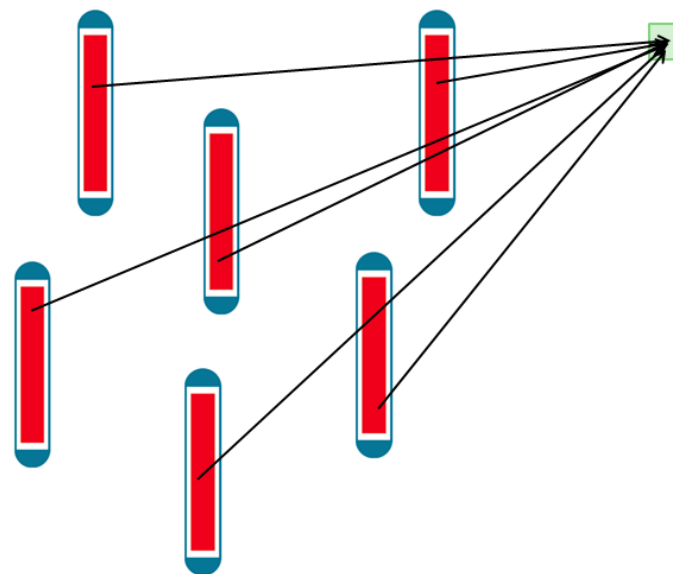
From top to bottom: Ring applicator, Bauer endometrial, and Joslin-Flynn Cervix applicator. [Reprinted with permission, Nucletron-Oldelft Corporation, Columbia, MD.]

TG43 – Limitations

Patient Size Effect



Interseed effects



TG43 – Limitations

TABLE I. Sensitivity of commonly treated anatomic sites to dosimetric limitations of the current brachytherapy dose calculation formalism. Items flagged as “Y” indicate the authors opinion that significant differences between administered and delivered dose are possible due to the highlighted dosimetric limitation.

Anatomic site	Source energy	Absorbed dose	Attenuation	Shielding	Scattering	Beta/kerma dose
Prostate	High	N	N	N	N	N
	Low	Y	Y	Y	N	N
Breast	High	N	N	N	Y	N
	Low	Y	Y	Y	N	N
GYN	High	N	N	Y	N	N
	Low	Y	Y	N	N	N
Skin	High	N	N	Y	Y	N
	Low	Y	N	Y	Y	N
Lung	High	N	N	N	Y	Y
	Low	Y	Y	N	Y	N
Penis	High	N	N	N	Y	N
	Low	Y	N	N	Y	N
Eye	High	N	N	Y	Y	Y
	Low	Y	Y	Y	Y	N

TG43 – Basic assumptions (...and limitations)

- Dose calculation in water
 - no material heterogeneities within the beam
 - no applicators
 - no shieldings
 - no source interactions

But very fast and in most cases accurate enough

- Inverse square law size effect
- Azimuthal symmetry

Alternative Approaches

The goal is to take

- patient inhomogeneities
- patient shape
- source interplays
- effects of applicators and shieldings

into account which are ignored by the TG43 protocol

- ➔ • Analytical Models (Convolution/Superposition, CC)
- Full Monte Carlo Simulations
- Deterministic solutions of the transport equations (LBTE)

AAPM TG 186

Main issues covered by the AAPM TG 186 report

- Limitations of the AAPM TG-43 formalism
- Review model-based brachytherapy dose calculation algorithms
- Dose report ($D_{m,m}$, $D_{w,m}$, D_{ref})
- CT imaging and patient motion
 - Media definition
 - Applicators, sources and other geometry
- MBDCA Commissioning
 - Level 1: MBDCA should fall back to TG-43 conditions (same as TG-43)
 - Level 2: MBDCA should take into account scatter conditions

Report of the Task Group 186 on model-based dose calculation methods in brachytherapy beyond the TG-43 formalism: Current status and recommendations for clinical implementation

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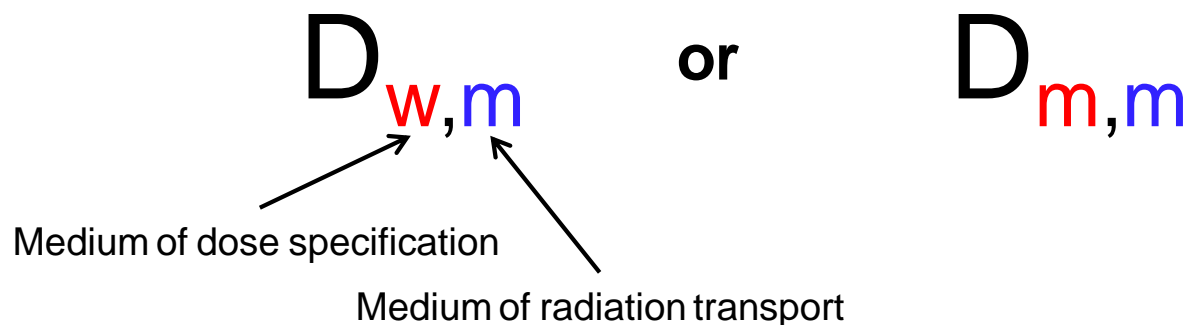
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published 25 September 2012)

The charge of Task Group 186 (TG-186) is to provide guidance for early adopters of model-based dose calculation algorithms (MBDCAs) for brachytherapy (BT) dose calculations to ensure practice uniformity. Contrary to external beam radiotherapy, heterogeneity correction algorithms have only recently been made available to the BT community. Yet, BT dose calculation accuracy is highly dependent on scatter conditions and photoelectric effect cross-sections relative to water. In specific situations, differences between the current water-based BT dose calculation formalism (TG-43) and MBDCAs can lead to differences in calculated doses exceeding a factor of 10. MBDCAs raise three major issues that are not addressed by current guidance documents: (1) MBDCAs calculated doses are sensitive to the dose specification medium, resulting in energy-dependent differences between dose calculated to water in a homogeneous water geometry (TG-43), dose calculated to the local medium in the heterogeneous medium, and the intermediate scenario of dose calculated to a small volume of water in the heterogeneous medium. (2) MBDCAs doses are sensitive to voxel-by-voxel interaction cross sections. Neither conventional single-energy CT nor ICRU/ICRP tissue composition compilations provide useful guidance for the task of assigning interaction cross sections to each voxel. (3) Since each patient-source-applicator combination is unique, having reference data for each possible combination to benchmark MBDCAs is an impractical strategy. Hence, a new commissioning process is required. TG-186 addresses in detail the above issues through the literature review

AAPM TG 186 – Dose Report



As available evidence does not directly support $D_{w,m}$, reporting $D_{m,m}$ is preferred as it is a conceptually well-defined quantity, in contrast to $D_{w,m}$, which is a theoretical construct without a physical realization in a nonwater medium. On the basis of all these considerations, it is the consensus of TG-186 to require only reporting of $D_{m,m}$ when using MBDCAs. This does not preclude individual practitioners or protocol groups from reporting $D_{w,m}$ or other quantities of interest along with $D_{m,m}$.

AAPM TG 186 – CT imaging and Patient modelling

- Consensus material definition

TABLE III. Material definitions. Water is given for comparison.

Tissue	% mass					Z > 8	Mass density g cm ⁻³
	H	C	N	O			
Prostate (Ref. 110)	10.5	8.9	2.5	77.4	Na(0.2), P(0.1), S(0.2), K(0.2)	1.04	
Mean adipose (Ref. 110)	11.4	59.8	0.7	27.8	Na(0.1), S(0.1), Cl(0.1)	0.95	
Mean gland (Ref. 110)	10.6	33.2	3.0	52.7	Na(0.1), P(0.1), S(0.2), Cl(0.1)	1.02	
Mean male soft tissue (Ref. 109)	10.5	25.6	2.7	60.2	Na(0.1), P(0.2), S(0.3), Cl(0.2), K(0.2)	1.03	
Mean female soft tissue (Ref. 109)	10.6	31.5	2.4	54.7	Na(0.1), P(0.2), S(0.2), Cl(0.1), K(0.2)	1.02	
Mean skin (Ref. 109)	10.0	20.4	4.2	64.5	Na(0.2), P(0.1), S(0.2), Cl(0.3), K(0.1)	1.09	
Cortical bone (Ref. 109)	3.4	15.5	4.2	43.5	Na (0.1), Mg (0.2), P (10.3), S (0.3), Ca(22.5)	1.92	
Eye lens (Ref. 109)	9.6	19.5	5.7	64.6	Na(0.1), P(0.1), S(0.3), Cl(0.1)	1.07	
Lung (inflated) (Ref. 109)	10.3	10.5	3.1	74.9	Na(0.2), P(0.2), S(0.3), Cl(0.3), K(0.2)	0.26	
Liver (Ref. 109)	10.2	13.9	3.0	71.6	Na(0.2), P(0.3), S(0.3), Cl(0.2), K(0.3)	1.06	
Heart (Ref. 109)	10.4	13.9	2.9	71.8	Na(0.1), P(0.2), S(0.2), Cl(0.2), K(0.3)	1.05	
Water	11.2			88.8		1.00	

Ref. 109: ICRU Report 46; Ref. 110: Woodard and White 1986

- Material assignment method

For a given organ, it is recommended that the tissue composition assignment be guided by contours approved by the radiation oncologist.

- CT artifact removal

If imaging artifacts are present, we recommend a complete override of the tissue in question with a uniform density, which may be the average of neighboring pixels.

Dose Calculation

- Point Source approximation
- AAPM-TG43
- **ACE (Collapsed Cone)**
- Full Monte Carlo
- Acuros (Boltzmann Solver)

Analytical Models - PSS Formalism

Dose is deposited locally through primary electrons set in motion by a photon interaction and a large fraction through scattered components.

- Separation of the primary dose and scattered dose components:

$$\mathbf{D} = \mathbf{D}_{\text{prim}} + \mathbf{D}_{1\text{sc}} + \mathbf{D}_{\text{msc}}$$

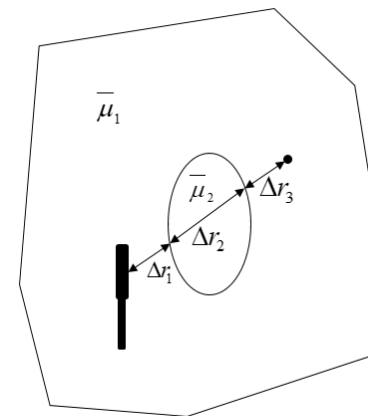
- New resampling of the TG43 data.

Analytical Models - PSS Formalism

$$D = D_{\text{prim}} + D_{\text{1sc}} + D_{\text{msc}}$$

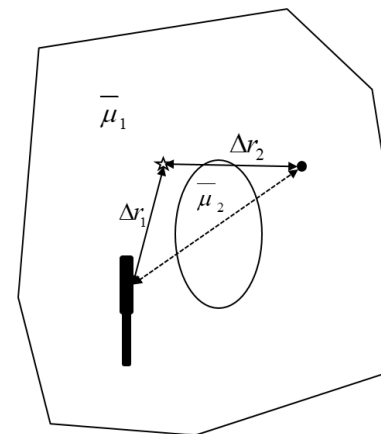
D_{prim}

- ➔ Phantom/patient size independent
- ➔ Energy dependent
- ➔ Source geometry dependent
- ➔ Depends on local mass attenuation coefficient (μ)

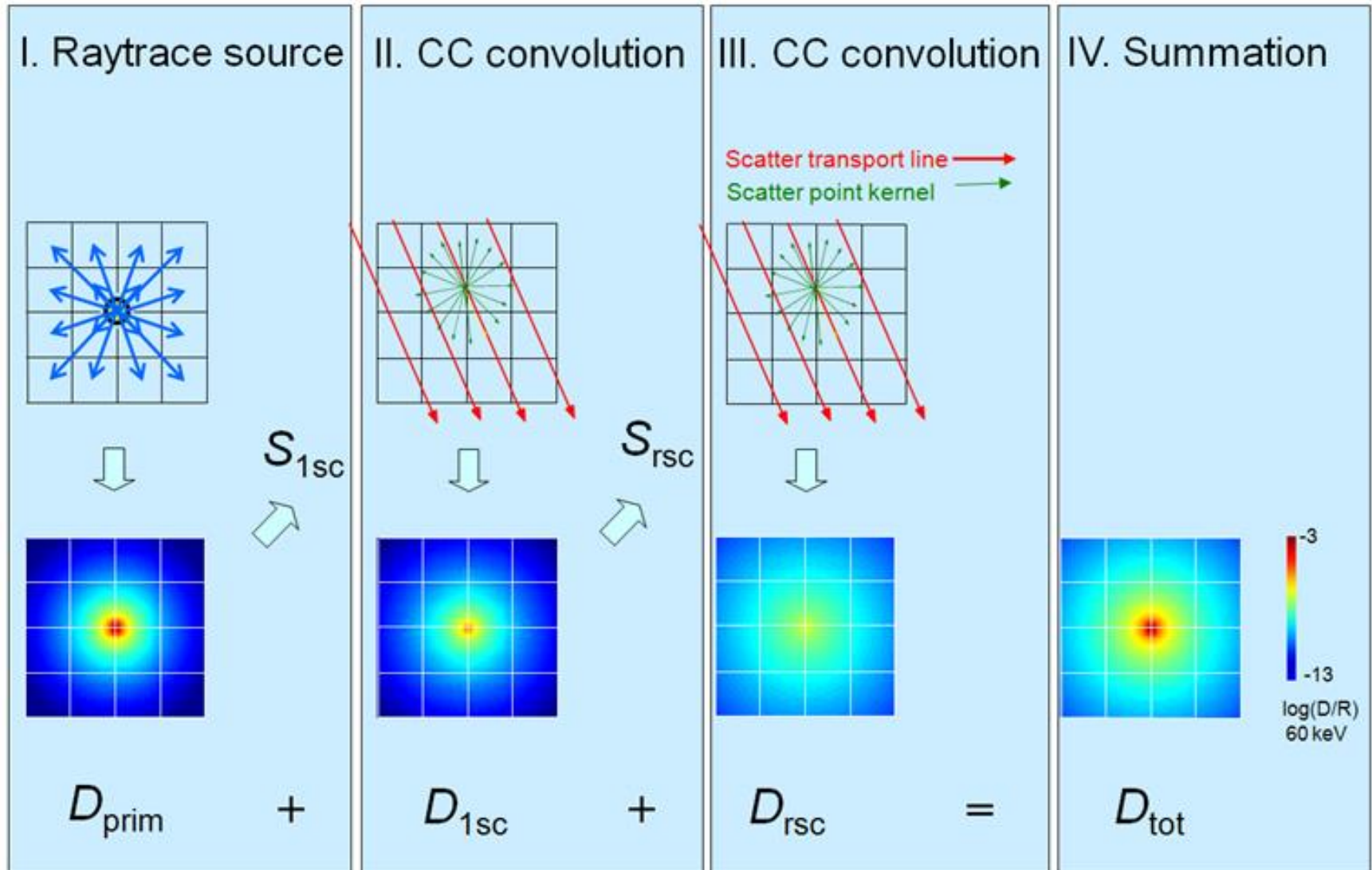


$D_{\text{1sc}} + D_{\text{msc}}$

- ➔ Depend on phantom/patient size
- ➔ Energy dependent
- ➔ Source geometry dependent
- ➔ Depend on primary dose (D_{prim})

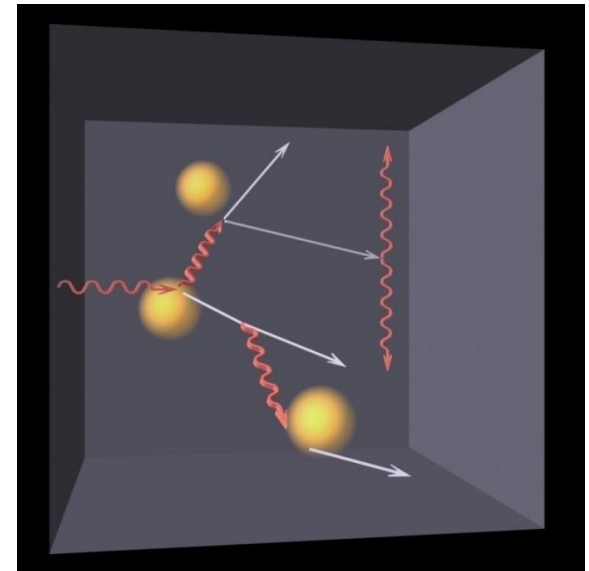


Analytical Models - Collapsed Cone



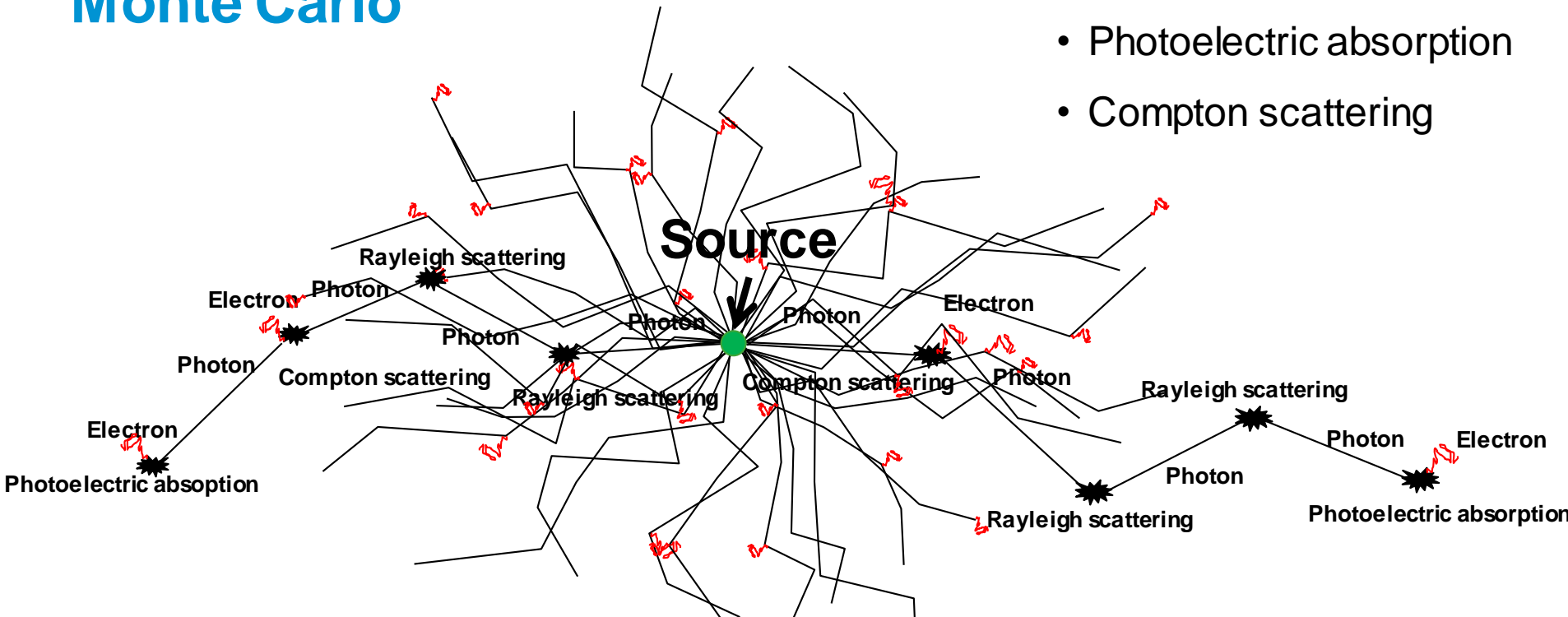
Dose Calculation

- Point Source approximation
- AAPM-TG43
- ACE (Collapsed Cone)
- **Full Monte Carlo**
- Acuros (Boltzmann Solver)



Monte Carlo

- Rayleigh scattering
- Photoelectric absorption
- Compton scattering



Dose per simulated history $\sim 10^{-13}$ Gy (Ir-192 point source)

➔ For 1 Gy about 10'000'000'000'000 histories needed!!

“Only” $\sim 1'000'000'000$ histories are simulated for statistically acceptable results

➔ Monte Carlo provide the user with an estimate of the solution.

Monte Carlo – Drawbacks in Brachytherapy

- Due to the $1/r^2$ behaviour of the primary dose component in brachytherapy, statistics rapidly degrade with increasing distance from the sources.
- Photons travel relatively long distances before interacting and their transport steps are often interrupted by boundary crossings.
- Elastic scattering does not deposit energy but still needs to be simulated.
Dose deposition needs many interaction steps.

Monte Carlo – Dose Deposition

Dose is deposited through electrons released by photon interactions. Due to the low energies in brachytherapy, the electron range is short. Thus, dose can be approximated as kerma (kinetic energy released per mass):

→ Dose ~ Kerma

$$D = \left(\frac{\mu_{en}}{\rho} \right)_m \Psi = K_{collision}$$

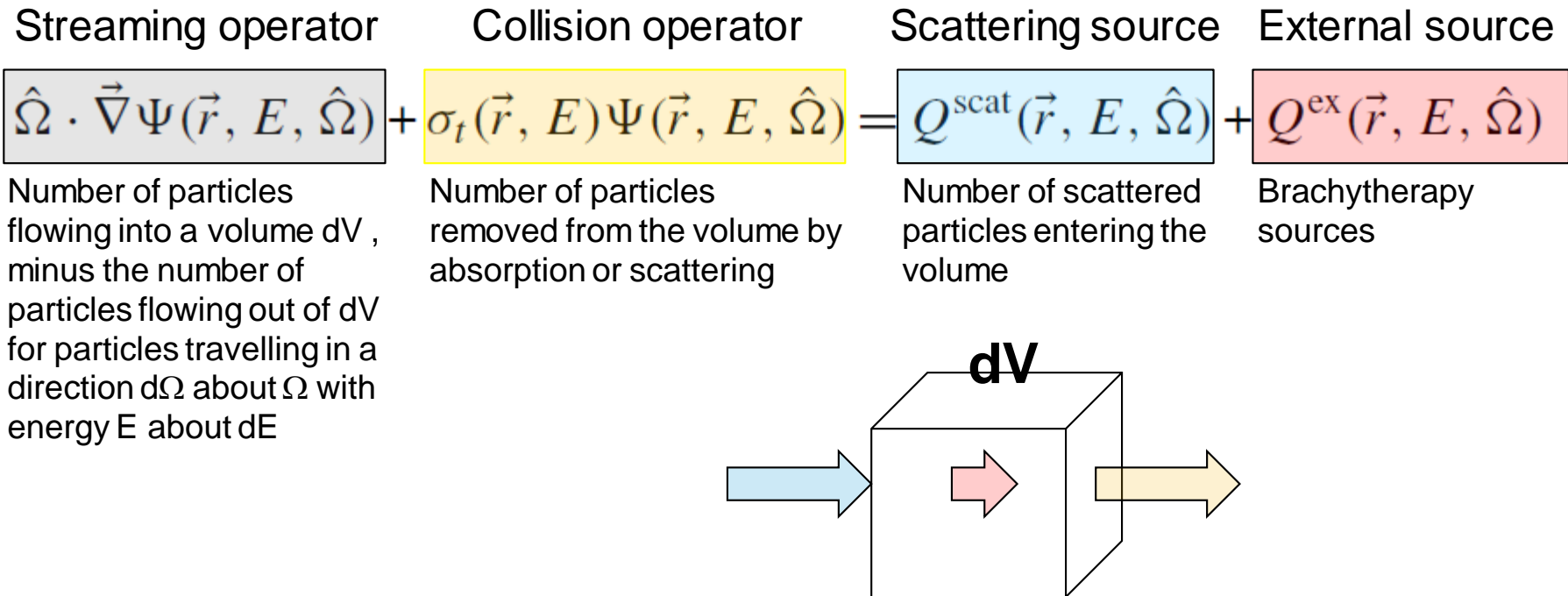
→ No electron transport

Dose Calculation

- Point Source approximation
- AAPM-TG43
- ACE (Collapsed Cone)
- Full Monte Carlo
- **Acuros (Boltzmann Solver)**

Tomorrow – Boltzmann Solver

An alternative approach is to solve the steady state Boltzmann transport equation in a Cartesian coordinate system:

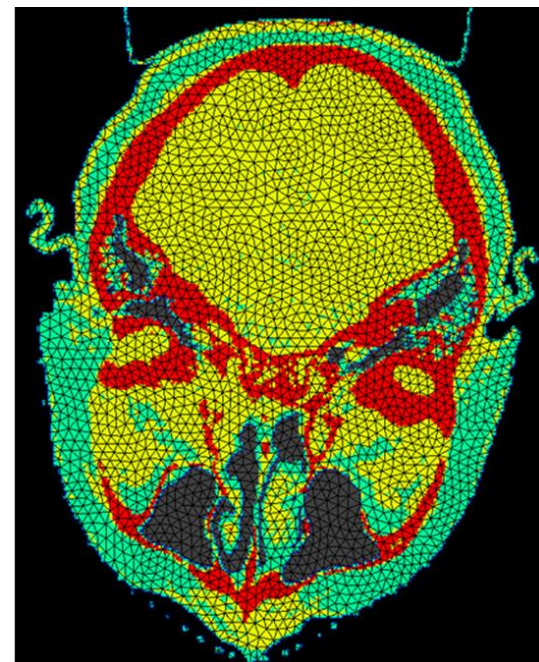


Ψ is the angular energy fluence at position $r = (x, y, z)$, with energy E , and direction $\Omega = (\mu, \eta, \zeta)$.

Tomorrow – Boltzmann Solver

The most common deterministic approach has been historically known as ‘discrete ordinates’:

- ➔ Discretization in space (finite element or finite difference), angle (discrete ordinates), and energy (multi-group cross sections)

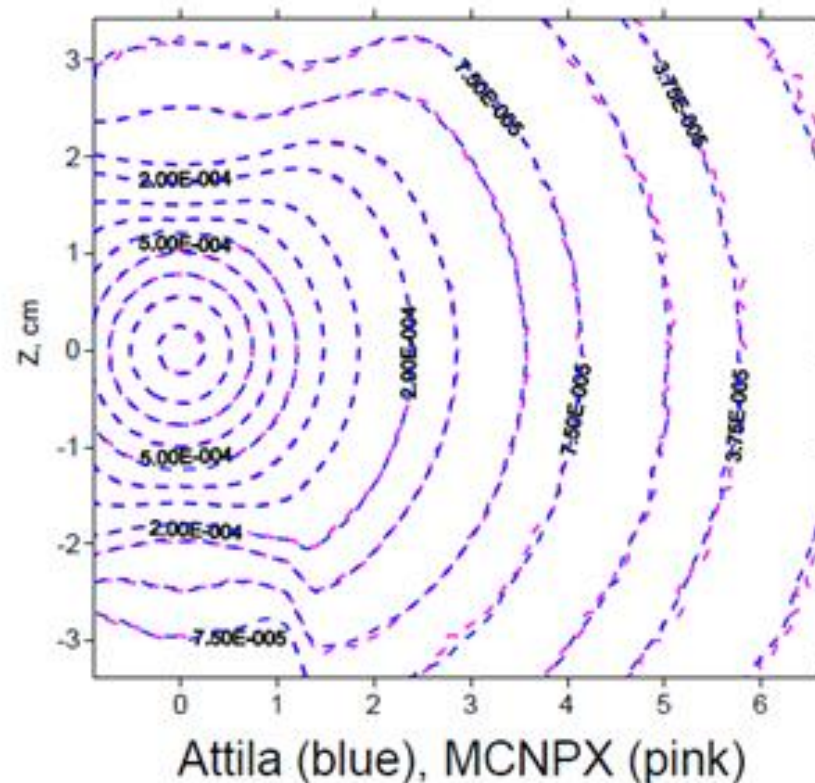
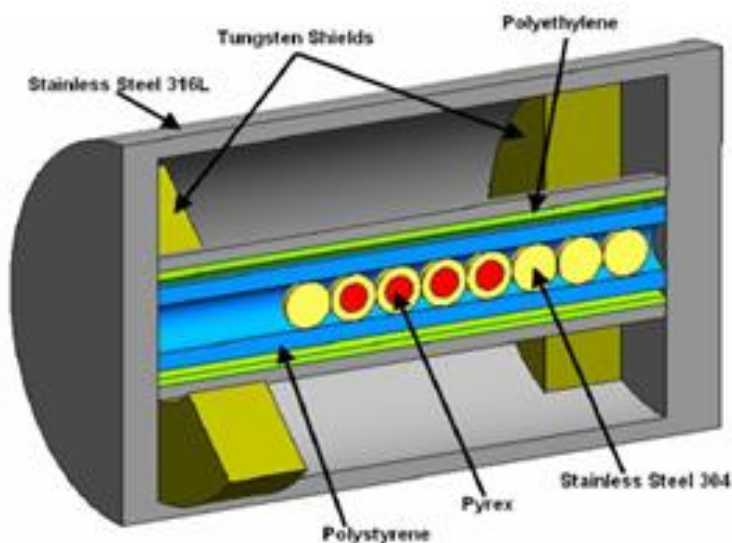


The challenge is to solve this equation for every sub-volume (dV) of the total volume (patient).

Tomorrow – Attila/Acuros™

F. Mourtada, T. Wareing, J. Horton, J. McGhee, D. Barnett, K. Gifford, G. Failla, R. Mohan, 'A Deterministic Dose Calculation Method Applied to the Dosimetry of Shielded Intracavitary Brachytherapy Applicators', *American Association of Physicists in Medicine Annual Meeting*, Pittsburgh, PA, 2004.

- ^{137}Cs pellets in tungsten shielded ovoid
- Comparison between Attila and Monte Carlo



Thank you

...Questions?