

APRENDE

Experiment: continuum range (GAINS@GELINA)

Insights in the GAINS data analysis procedure

26 - 27/02/2025
APRENDE Experimentalists-Evaluators workshop (WP2-WP4)

Adina Coman, IFIN-HH

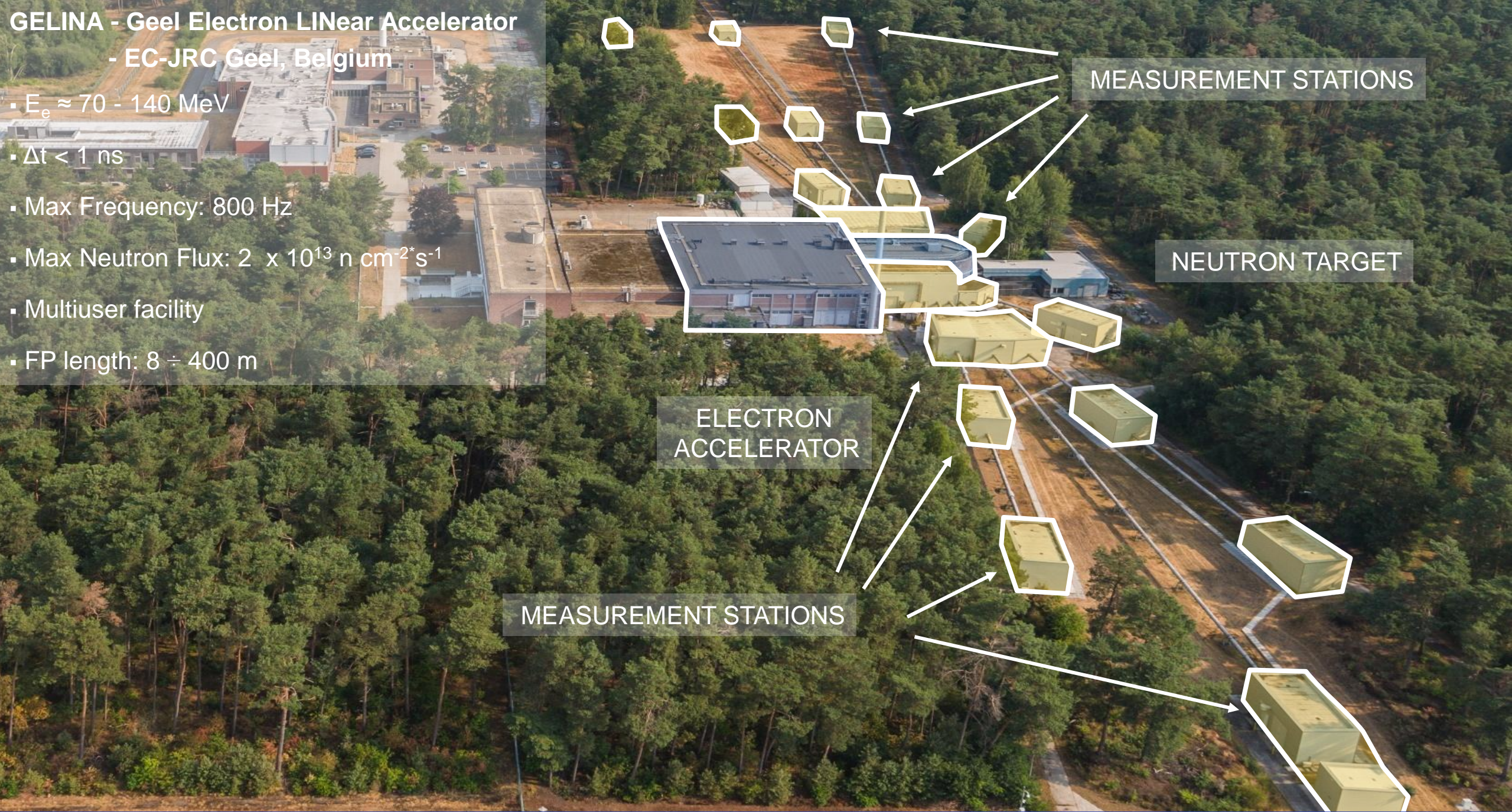


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GELINA - Geel Electron LINear Accelerator - EC-JRC Geel, Belgium

- $E_e \approx 70 - 140 \text{ MeV}$
- $\Delta t < 1 \text{ ns}$
- Max Frequency: 800 Hz
- Max Neutron Flux: $2 \times 10^{13} \text{ n cm}^{-2}\text{s}^{-1}$
- Multiuser facility
- FP length: $8 \div 400 \text{ m}$



MEASUREMENT STATIONS

NEUTRON TARGET

ELECTRON
ACCELERATOR

MEASUREMENT STATIONS



GAINS Spectrometer

Gamma Array for Inelastic Neutron Scattering

- 12 large-volume HPGe detectors:

- 4 @ 110°
- 4 @ 150°
- 4 @ 125°

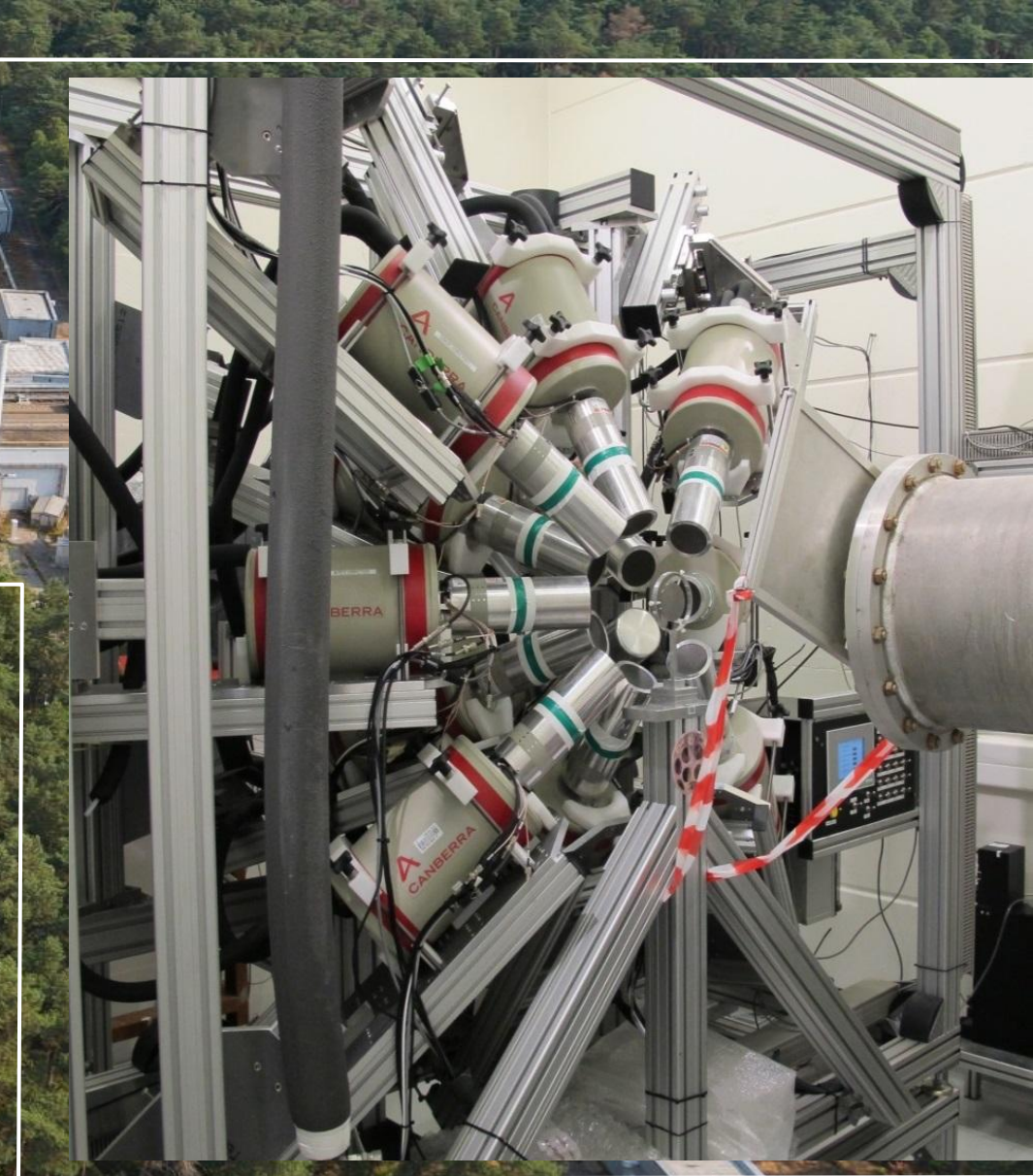
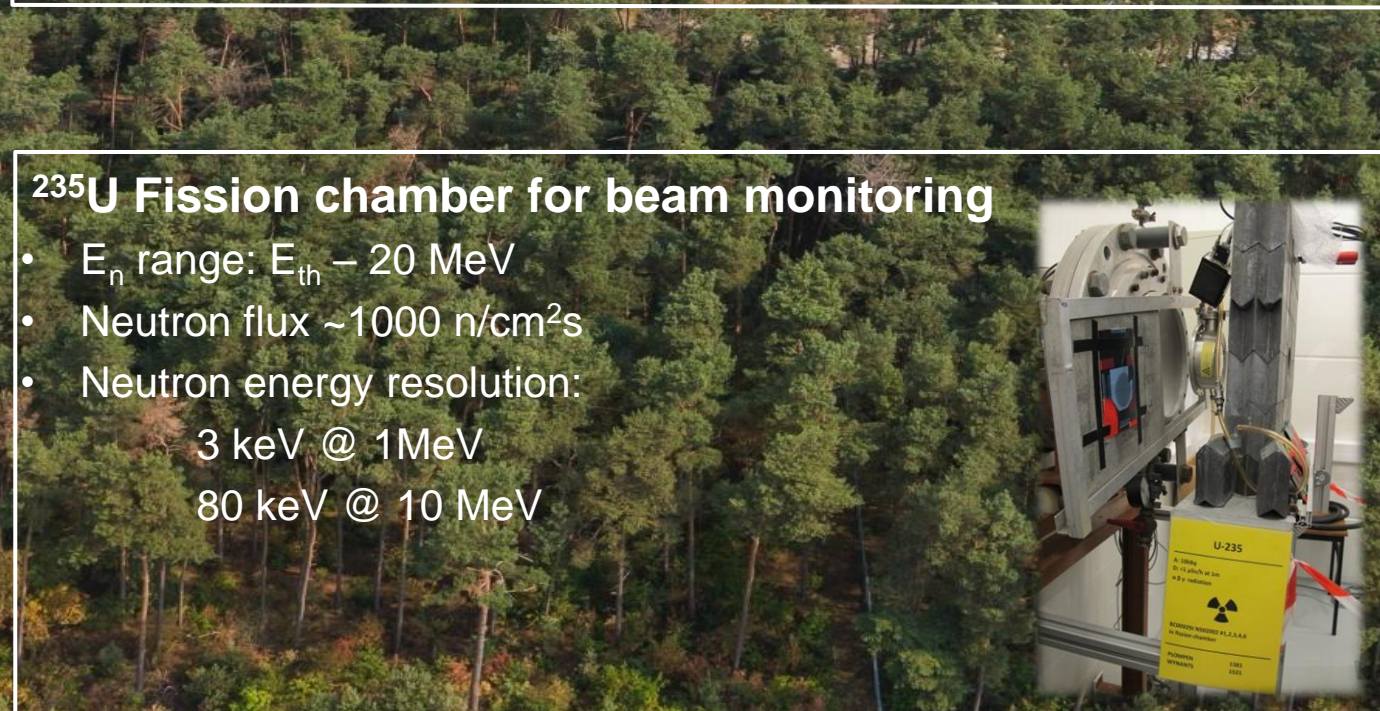
100% relative efficiency

2.3 keV energy resolution for 1332 keV of ^{60}Co



^{235}U Fission chamber for beam monitoring

- E_n range: E_{th} – 20 MeV
- Neutron flux ~ 1000 n/cm 2 s
- Neutron energy resolution:
 - 3 keV @ 1 MeV
 - 80 keV @ 10 MeV





Cross sections for inelastic scattering of neutrons on ²⁸Si and comparison with the ²⁵Mg(α, n)²⁸Si reaction

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 (Received 29 May 2013; published 11 September 2013)

The inelastic scattering of neutrons on ²⁸Si was investigated using the $(n, n'\gamma)$ technique. The γ production cross sections were measured and the level and total inelastic cross sections were determined with high accuracy. ²⁸Si was also excited through the ²⁵Mg($\alpha, n\gamma$)²⁸Si reaction and a comparison of the γ production cross sections determined for various incident energies was performed.

PHYSICAL REVIEW C 90, 034603 (2014)

Neutron inelastic cross-section measurements for ²⁴Mg

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 (Received 24 June 2014; published 8 September 2014)

The γ production cross sections from neutron inelastic scattering on ²⁴Mg were measured for neutron up to 18 MeV at GELINA (the Geel Linear Accelerator), the neutron source operated by EC-JRC Belgium. The level cross section and the total inelastic cross section were determined. We used the (Gamma Array for Inelastic Neutron Scattering) spectrometer with seven large-volume high-purity germanium detectors placed at 110° and 150° with respect to the beam direction. The neutron flux was determined by the ²³⁵U fission chamber.

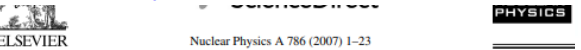
PHYSICAL REVIEW C 96, 014621 (2017)

Neutron inelastic scattering measurements on the stable isotopes of titanium

A. Olacel,^{1,*} F. Belloni,² C. Borcea,¹ M. Boromiza,^{1,3} P. Dessagne,⁴ G. Henning,⁴ M. Kerveno,⁴ A. E. Pirovano,² and A. J. M. Plompen²
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 (Received 24 May 2017; published 31 July 2017)

The results of a neutron inelastic scattering experiment performed at the Geel Electron Linear Accelerator (GELINA) are reported. The neutron source of the European Commission Joint Research Centre is used. The γ production cross sections for incident neutrons up to 18 MeV were measured. The level and total inelastic cross sections were determined. Our experiment is compared with theoretical calculations performed using the TALYS 1.8 code, evaluated nuclear data libraries, and previously reported results.

DOI: 10.1103/PhysRevC.96.014621



PHYSICS

High resolution measurement of neutron inelastic scattering and $(n, 2n)$ cross-sections for ⁵²Cr

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 Received 17 November 2006; received in revised form 15 December 2006; accepted 8 January 2007

Available online 12 January 2007

Cross section measurements for neutron inelastic scattering and the $(n, 2n)$ reaction on ²⁰⁶Pb

A. Negret,^{1,*} L. C. Mihailescu,^{1,2} C. Borcea,¹ Ph. Dessagne,^{3,4} K. H. Guber,⁵ M. Kerveno,^{3,4} A. J. Koning,⁶ A. Olacel,^{1,2} A. J. M. Plompen,² C. Rouki,² and G. Rudolf^{3,4}
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⁴CNRS, UMR7178, F-67037 Strasbourg, France
⁵Reactor and Nuclear Systems Division, Oak Ridge National Laboratory, Oak Ridge, Tennessee 37831, USA
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 (Received 27 May 2015; published 30 June 2015)

Excitation functions for γ production associated with the neutron inelastic scattering and the $(n, 2n)$ reactions on ²⁰⁶Pb were measured from threshold up to 18 MeV for about 40 transitions. Two independent measurements were performed using different samples and acquisition systems to check consistency of the results. The neutron flux was determined with a ²³⁵U fission chamber and a procedure that were validated against a fluence standard. For incident energy higher than the threshold for the first excited level and up to 3.5 MeV, estimates are provided for the total inelastic and level cross sections by combining the present γ production cross sections with the level and decay data of ²⁰⁶Pb reported in the literature. The uncertainty common to all incident energies is 3.0% allowing overall uncertainties from 3.3% to 30% depending on transition and neutron energy. The present data agree well with earlier work, but significantly expand the experimental database while comparisons with model calculations are shown.

PHYSICAL REVIEW C 93, 024610 (2016)

Inelastic scattering cross section following inelastic scattering by ⁷Li

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PHYSICAL REVIEW C 106, 024609 (2022)

Nucleon-induced inelastic cross sections on ^{nat}Ni

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⁵Physics Department, University Politehnica of Bucharest, Splaiul Independentei Number 313, 060042 Bucharest-Sector 6, Romania
 (Received 8 June 2022; accepted 26 July 2022; published 17 August 2022)

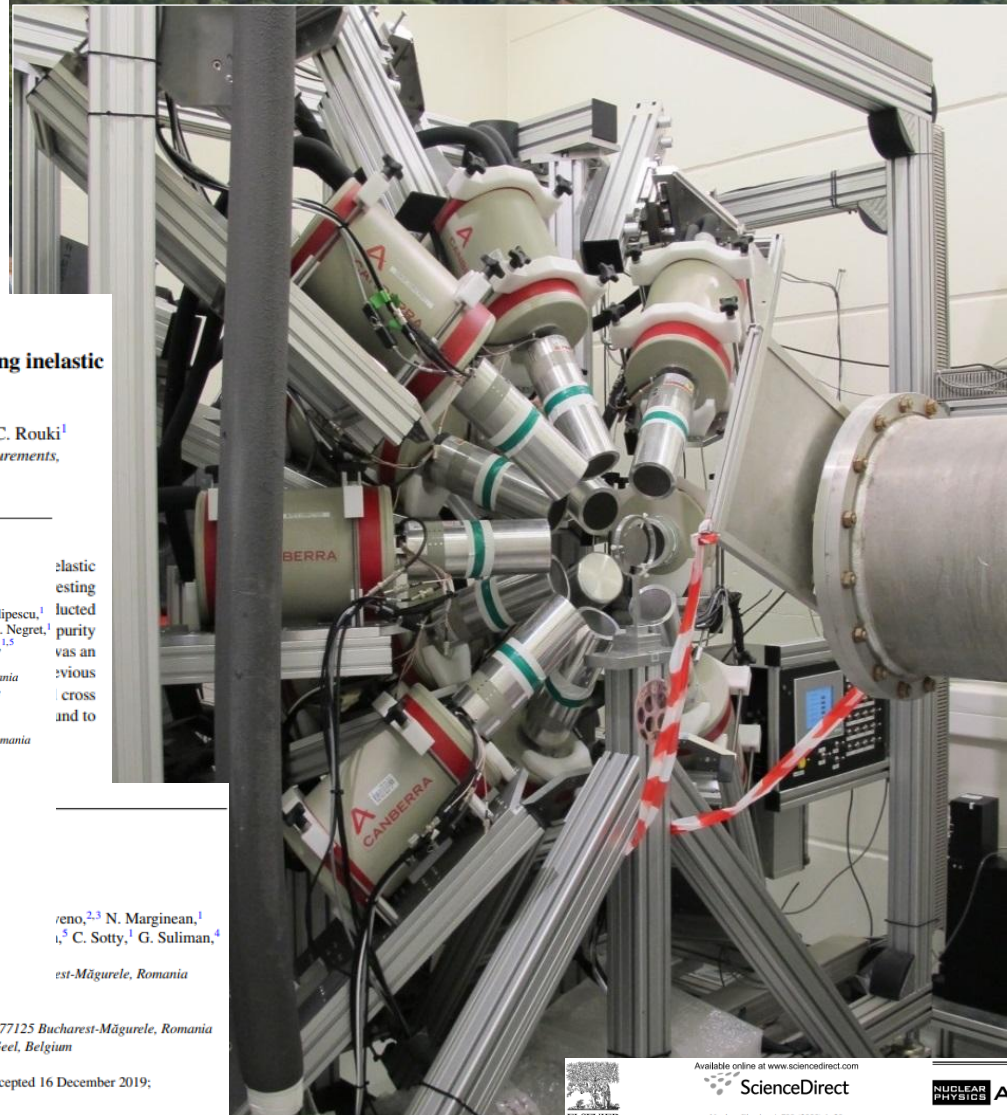
This paper reports on the results of $(n, n'\gamma)$ and $(p, p'\gamma)$ cross section measurements on nickel performed at the Geel Electron Linear Accelerator of the European Commission, Joint Research Centre (Geel) and at the 9-MV Tandem Accelerator of Horia Hulubei National Institute for Physics and Nuclear Engineering (Bucharest-Măgurele), respectively. The main goal was to reliably measure with small uncertainty the most intense transitions arising from the inelastic channel. Comparisons are performed between the extracted results, nuclear reaction model calculations using default parameter values, and previously reported measurements, if available. The broader goal of this paper is related to our study on the possibility of inferring neutron inelastic cross sections from the proton-induced ones, in this case for ^{nat}Ni. We show that—by making use of the Lane consistency of the nucleon optical model potential and of the constraints offered by the proton data—one can extract a neutron-target potential that better describes the experimental data, as compared to the calculation with default neutron parameters. We also discuss relevant issues and still open questions of our calculations along with future plans for mitigation.

DOI: 10.1103/PhysRevC.106.024609

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(Received 13 September 2019; revised manuscript received 25 November 2019; accepted 16 December 2019; published 10 February 2020)

This paper reports cross-section measurements of the (n, n') and (p, p') reactions on ¹⁶O and ²⁸Si at Geel Electron Linear Accelerator and at the 9-MV Tandem Accelerator of Horia Hulubei National Institute for Physics and Nuclear Engineering, respectively. The main purpose was to measure the neutron- and proton-induced inelastic γ -production cross sections for all observed transitions in ¹⁶O and ²⁸Si, followed by the calculation of the corresponding total inelastic cross section. The results are compared with theoretical calculations performed using the TALYS 1.9 code, evaluated nuclear data, and previously reported experimental data. The broader goal of this work is to study whether and to what extent the neutron-induced inelastic cross sections of these nuclei can be inferred from those obtained using suitable charged particle reactions. We show that, by making use of the formal similarities between the neutron- and the proton-target optical model potentials and isospin symmetry in mirror nuclei, one can develop a procedure that combines experimental proton-induced inelastic cross sections with theoretical calculations to infer neutron inelastic cross sections. For ¹⁶O and ²⁸Si, the precision associated with this procedure is around 10–20% for most of the incident energy range.



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eno,^{2,3} N. Marginean,¹
 i,³ C. Sotty,¹ G. Suliman,⁴
 est-Măgurele, Romania

PHYSICAL REVIEW C 90, 034602 (2014)

Cross-section measurements for the ⁶⁶Fe($n, xn\gamma$) reactions

A. Negret,^{1,*} C. Borcea,¹ Ph. Dessagne,² M. Kerveno,² A. Olacel,^{1,3} A. J. M. Plompen,⁴ and M. Stanoiu¹
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 (Received 25 June 2014; revised manuscript received 12 August 2014; published 5 September 2014)

A measurement was performed at the white neutron source Geel Linear Accelerator (GELINA) of the Institute for Reference Materials and Measurements to determine the cross sections for the $(n, n'\gamma)$ and $(n, 2n'\gamma)$ reactions on ⁶⁶Fe. The Gamma Array for Inelastic Neutron Scattering (GAINS) was used. The results are scaled to the neutron-induced fission cross section of ²³⁵U. The paper emphasizes the multiple checks performed to assure the consistency of the results: γ production cross sections, total inelastic cross sections, and level production cross sections were determined. A good agreement exists with previous measurements. Theoretical calculations were performed with the TALYS 1.6 reaction code using default parameters, but also using a microscopic approach. These calculations are discussed in comparison to the experimental data.

DOI: 10.1103/PhysRevC.90.034602

PACS numbers: 25.40.Fq, 29.30.Kv, 27.40.+z



Nuclear Physics A 799 (2008) 1–29

Measurement of neutron inelastic scattering and $(n, 2n)$ cross-sections for ²⁰⁹Pb

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^d Faculty of Physics, University of Vienna, Währinger Straße 17, 1090 Wien, Austria

Data analysis procedure

- γ spectroscopy measurements
- Differential γ -production cross sections @ 110° & 150°: primary results

$$\frac{d\sigma}{d\Omega}(\theta_i, E_k) = \frac{1}{4\pi} \frac{Y_j(E_k)}{Y_{FC}(E_k)} \frac{\epsilon_{FC}}{\epsilon_j} \frac{\sigma_U(E_k)}{t_s} \frac{t_U A_S}{A_U} \frac{1}{c_{ms}(E_k)}$$

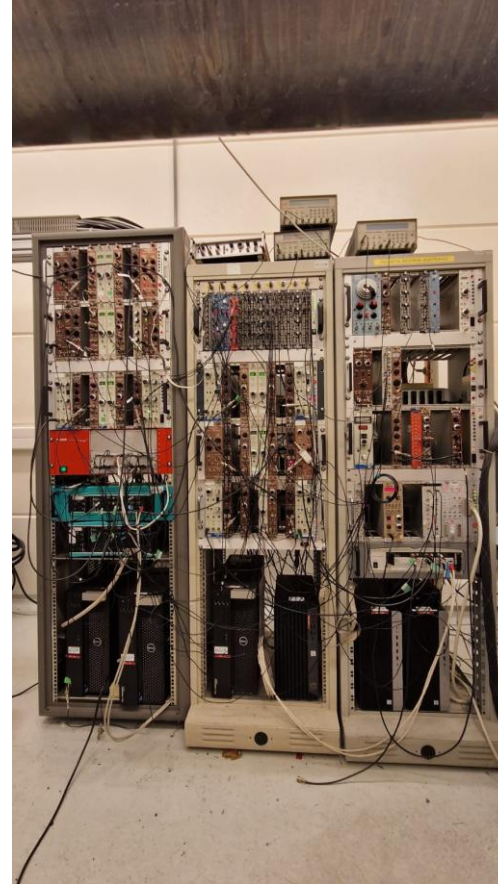
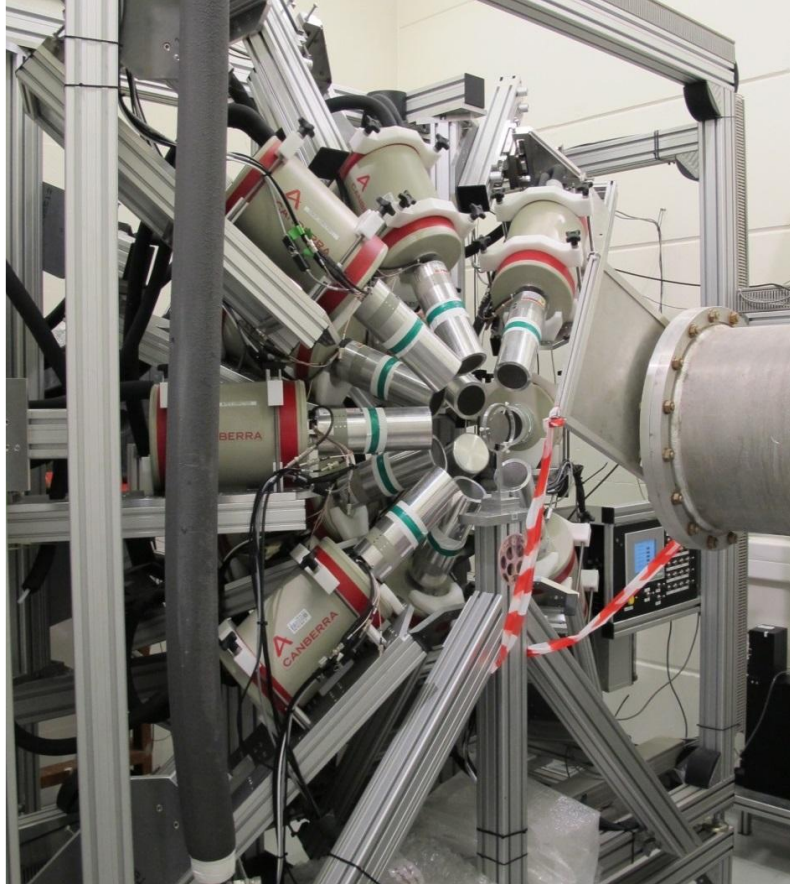
Data analysis procedure

HPGE detectors's data reduction



➤ Differential γ -production cross sections

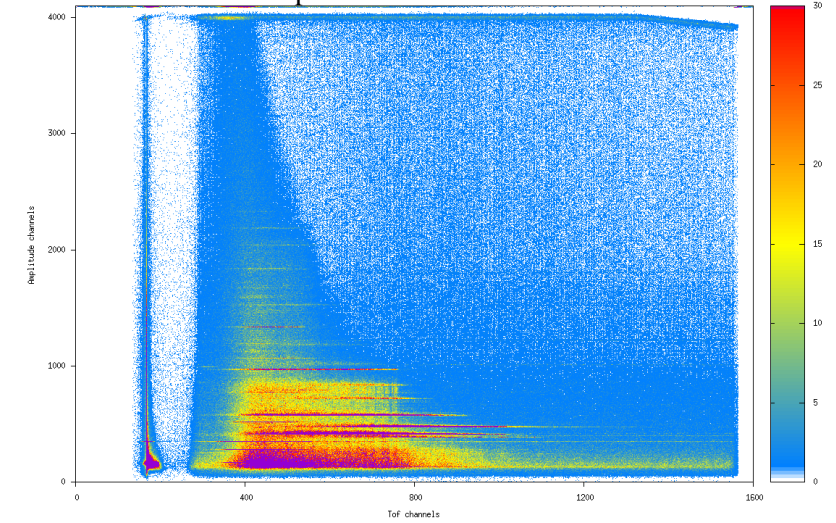
$$\frac{d\sigma}{d\Omega}(\theta_i, E_k) = \frac{1}{4\pi} \frac{Y_j(E_k)}{Y_{FC}(E_k)} \frac{\varepsilon_{FC} \sigma_U(E_k)}{\varepsilon_j} \frac{t_U A_S}{t_S A_U} \frac{1}{c_{ms}(E_k)}$$



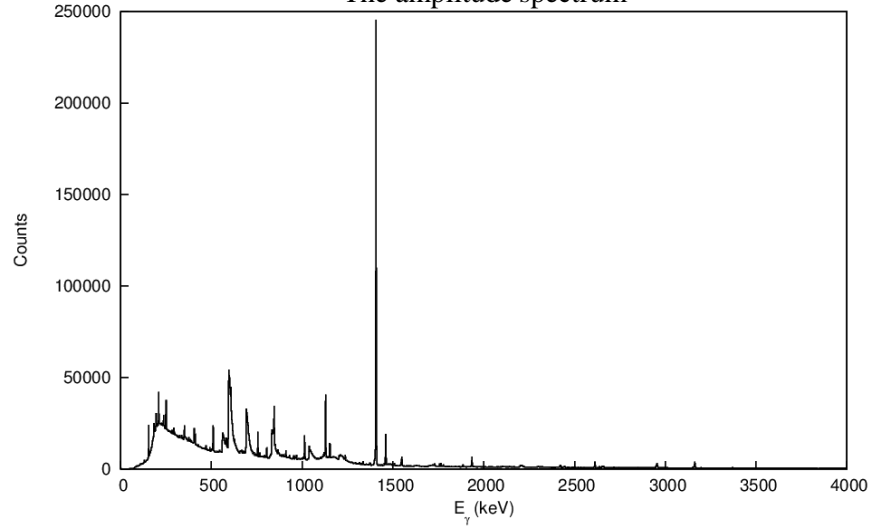
➤ Time-amplitude listfiles recorded for 2h



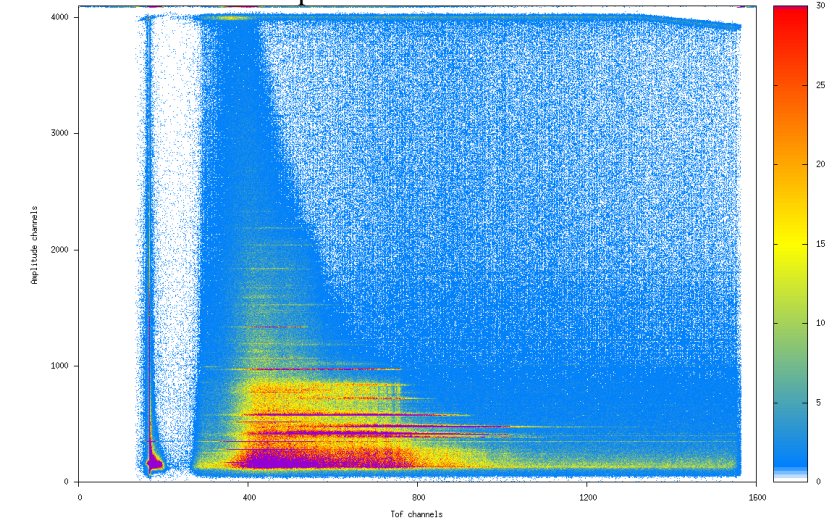
The time-amplitude matrix of one HPGe detector



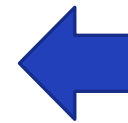
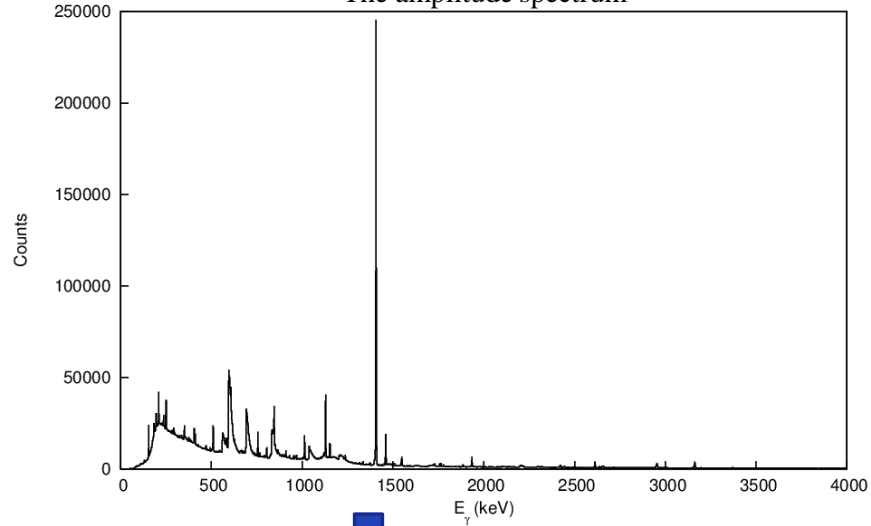
The amplitude spectrum



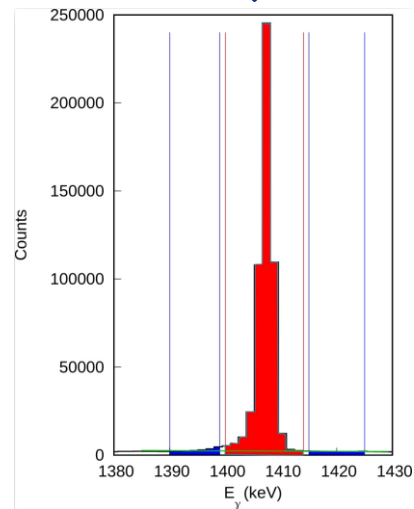
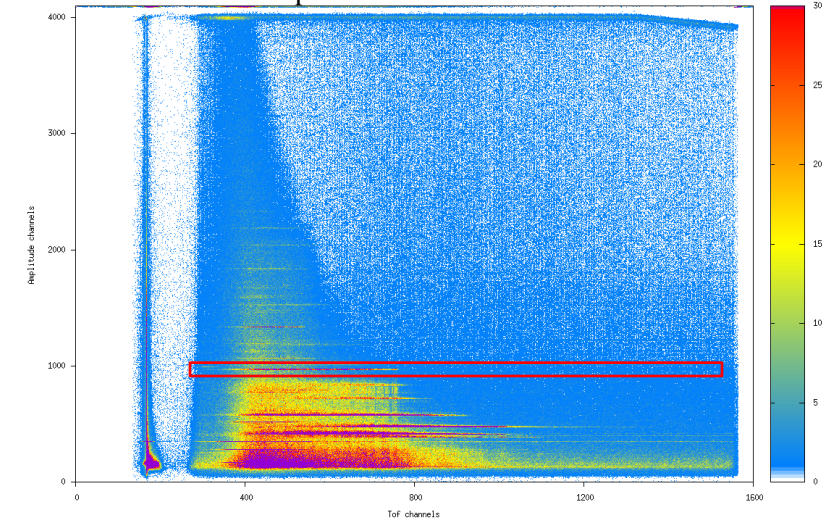
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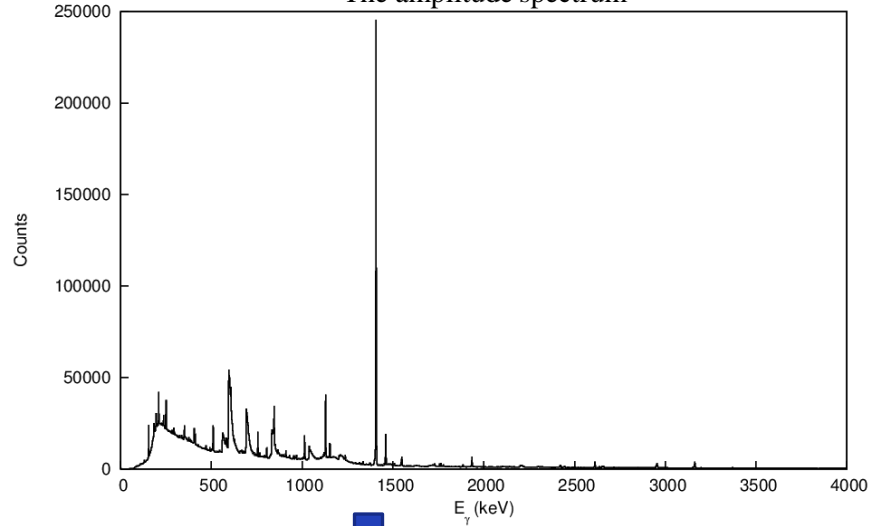
The amplitude spectrum



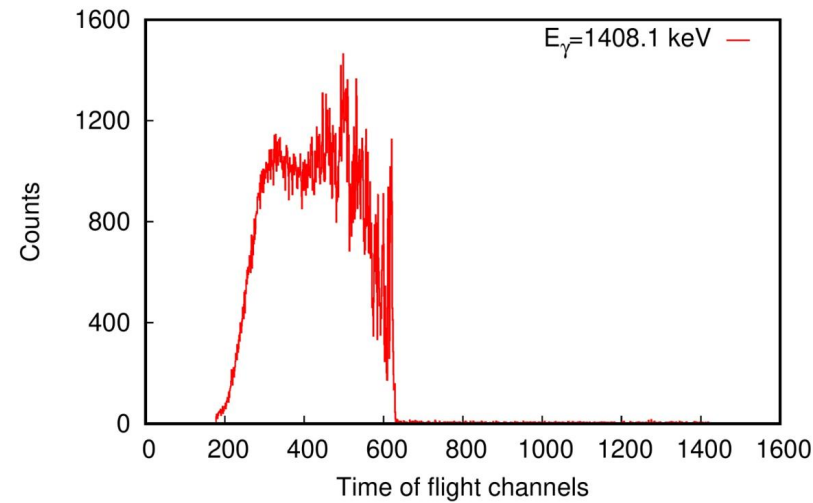
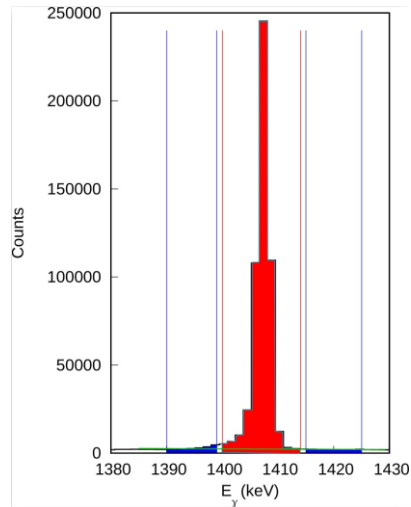
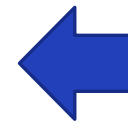
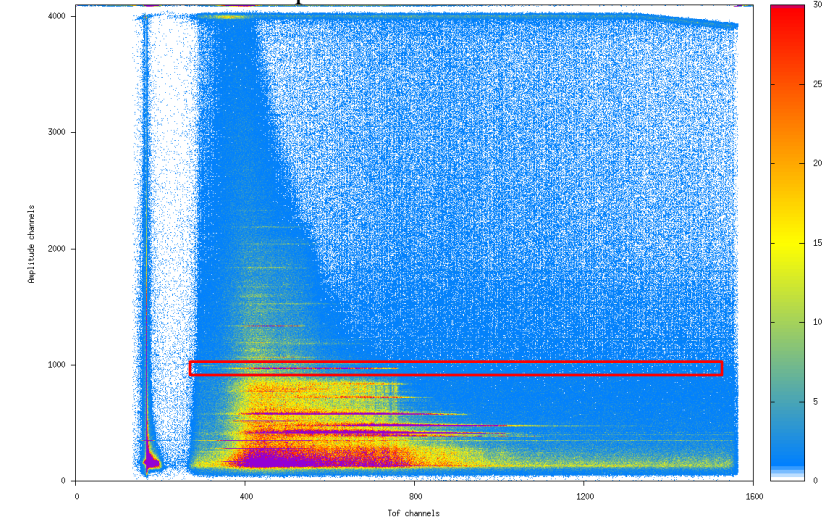
The time-amplitude matrix of one HPGe detector

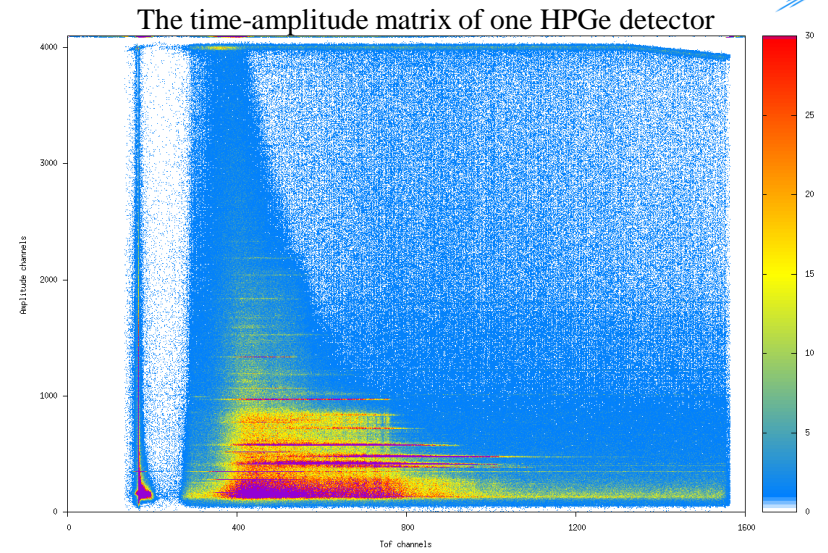


The amplitude spectrum



The time-amplitude matrix of one HPGe detector

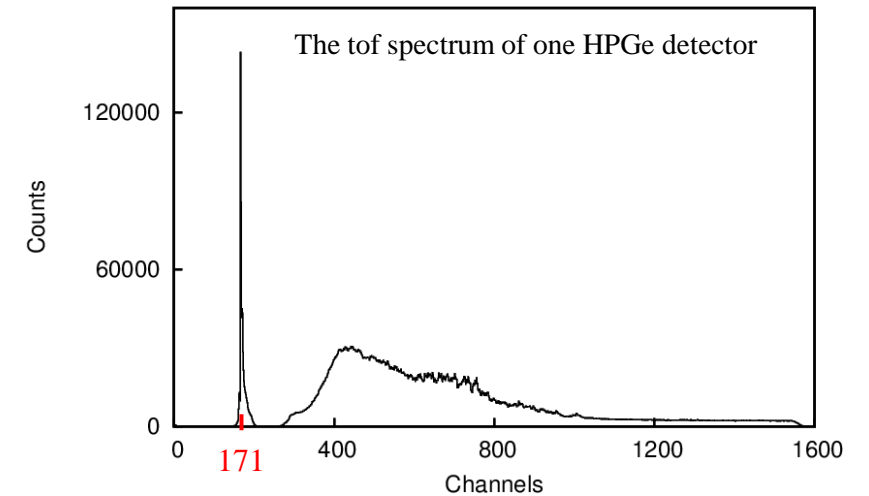
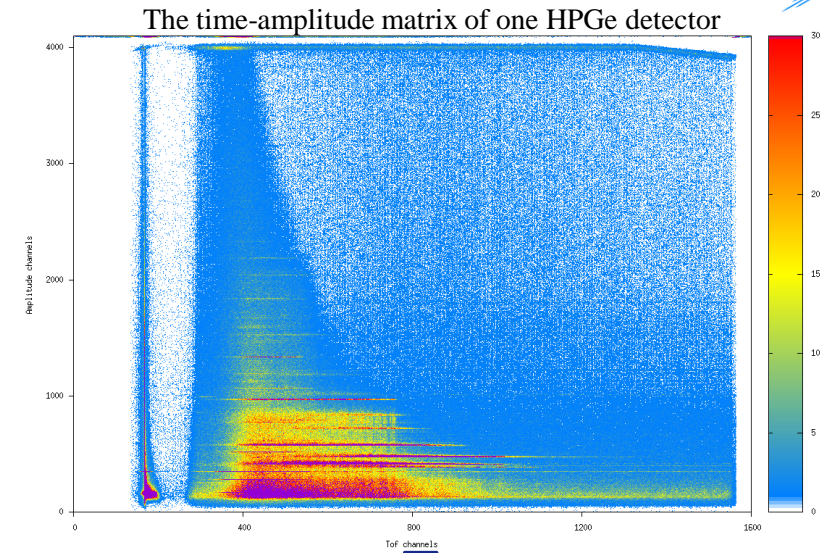
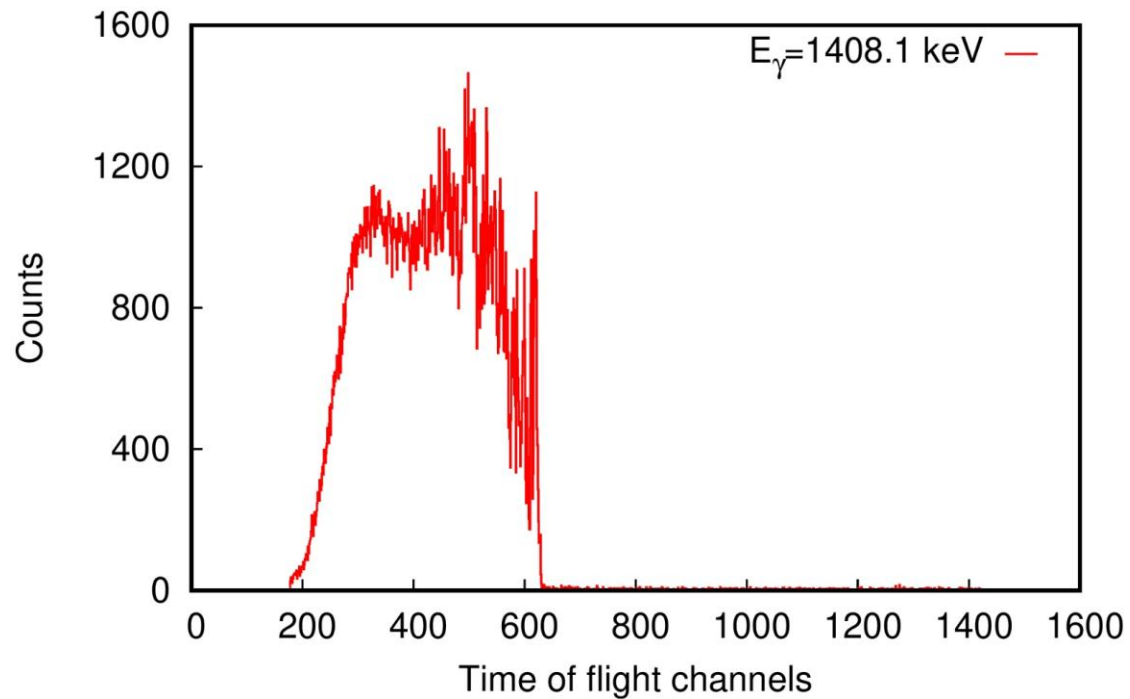




$$t_{\gamma\text{-flash}} = \frac{d_{\text{flight path}}}{c} = \frac{9967.55 \text{ cm}}{29.979 \text{ cm/ns}} = 332.484 \text{ ns}$$

➤ calibration in time of flight

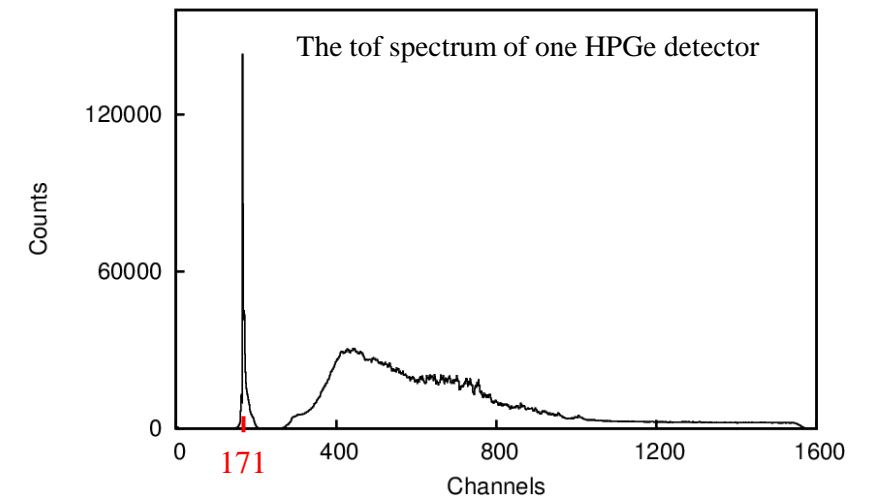
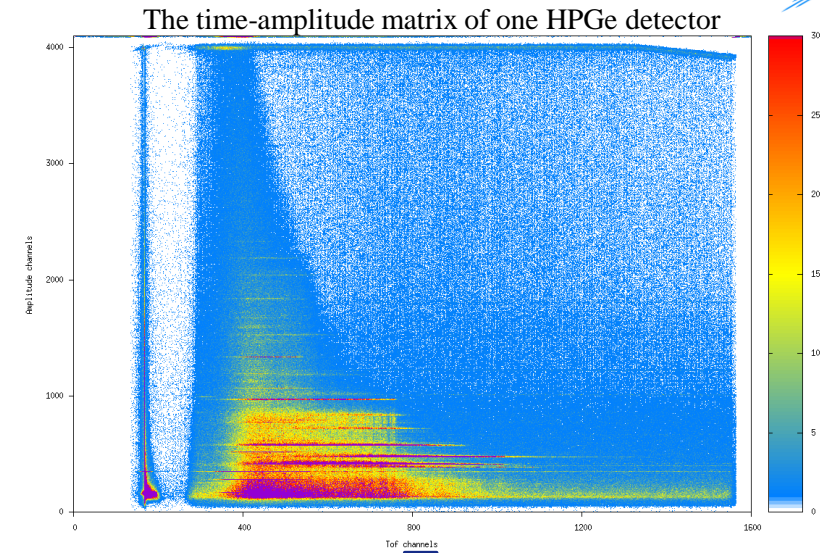
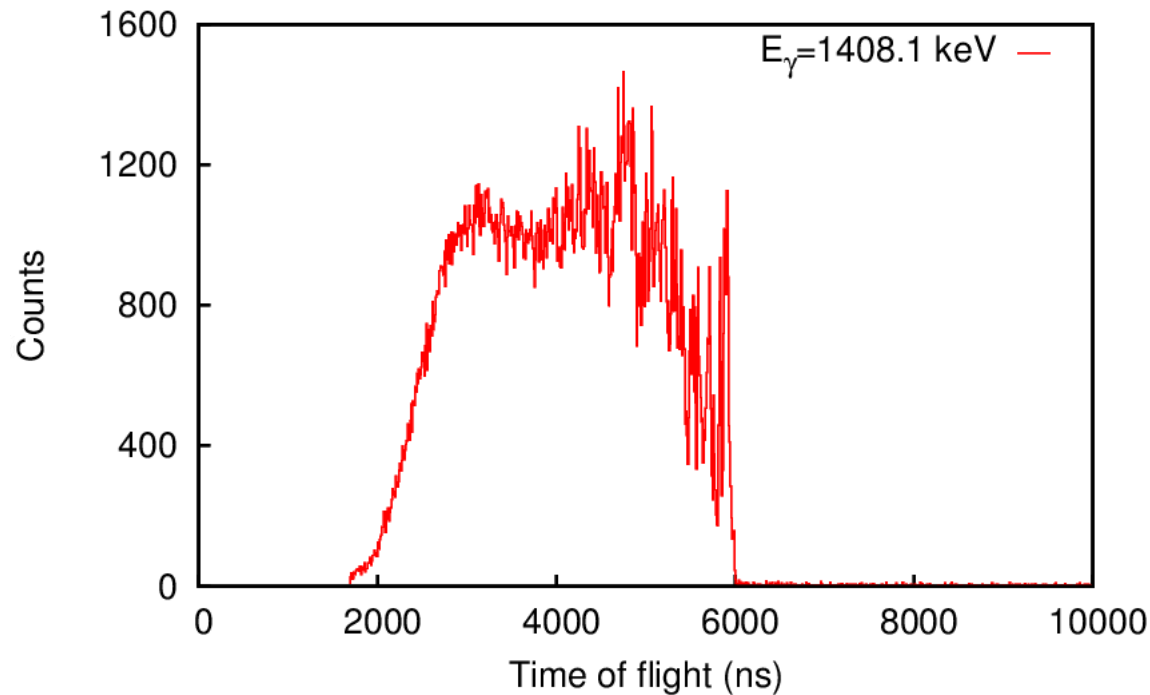
$$\text{tof}(ns) = [\text{tof}(\text{channels}) - \gamma\text{-flash}(\text{channels})] * 9.5238 \text{ ns} - t_{\gamma\text{-flash}}$$



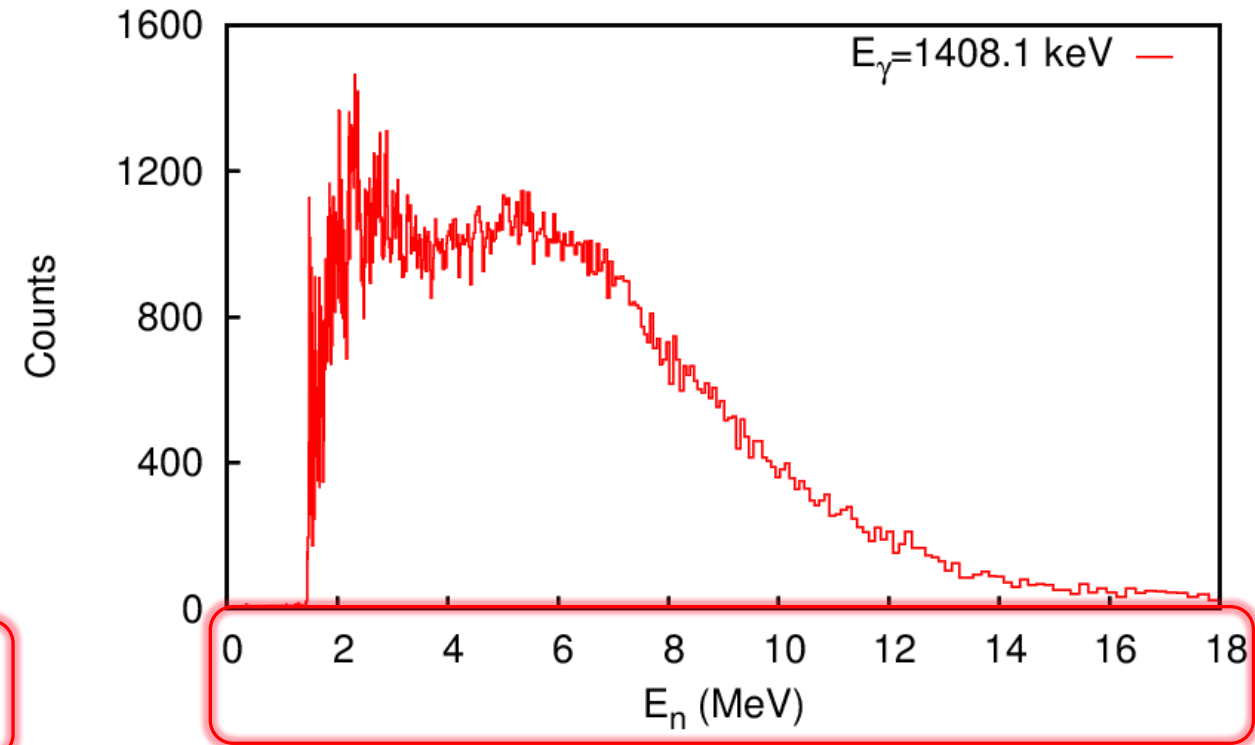
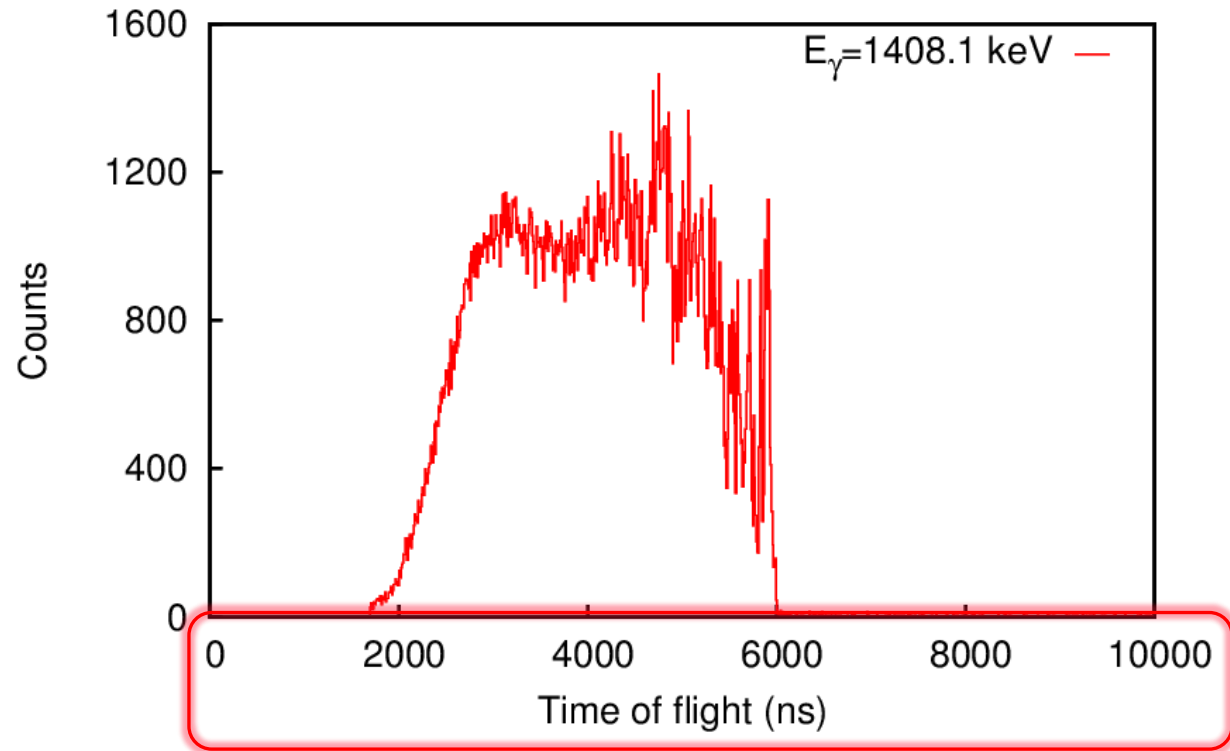
$$t_{\gamma\text{-flash}} = \frac{d_{\text{flight path}}}{c} = \frac{9967.55 \text{ cm}}{29.979 \text{ cm/ns}} = 332.484 \text{ ns}$$

➤ calibration in time of flight

$$\text{tof}(\text{ns}) = [\text{tof}(\text{channels}) - \gamma\text{-flash}(\text{channels})] * 9.5238 \text{ ns} - t_{\gamma\text{-flash}}$$



$$E_n = m_0 c^2 \left[\frac{1}{\sqrt{1 - \frac{v^2}{c^2}}} - 1 \right] \quad \& \quad \text{ToF-tehnique}$$



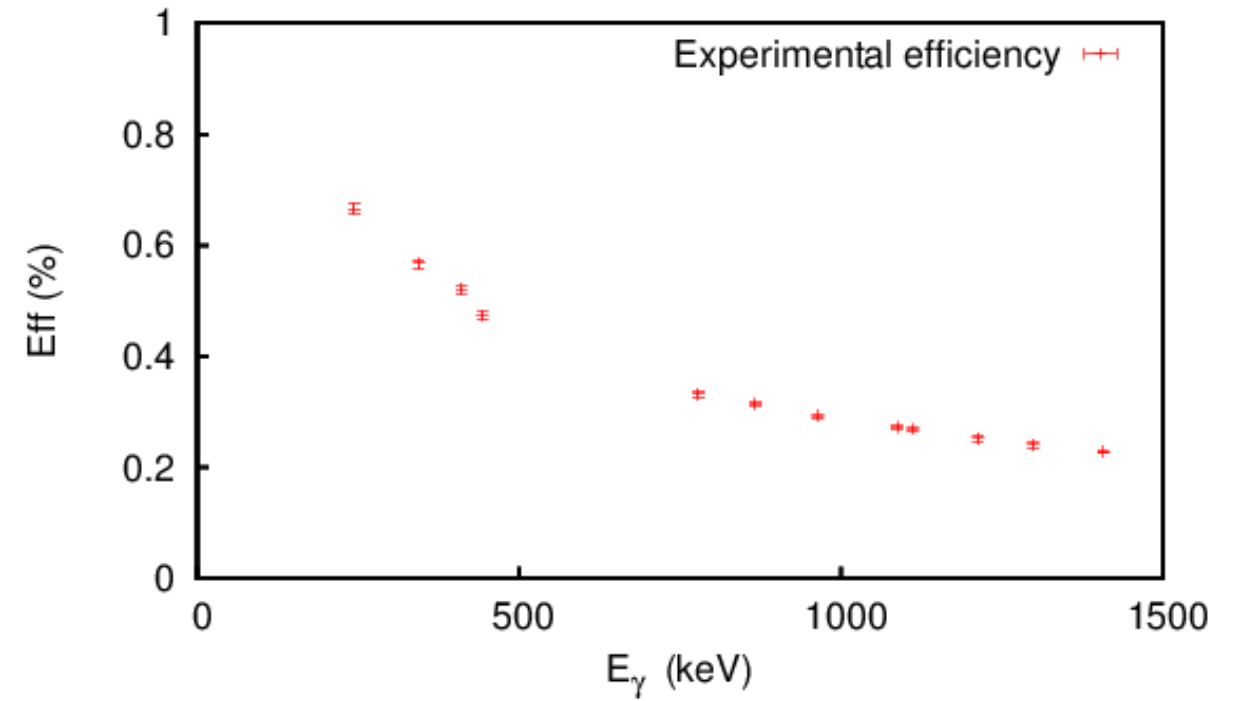
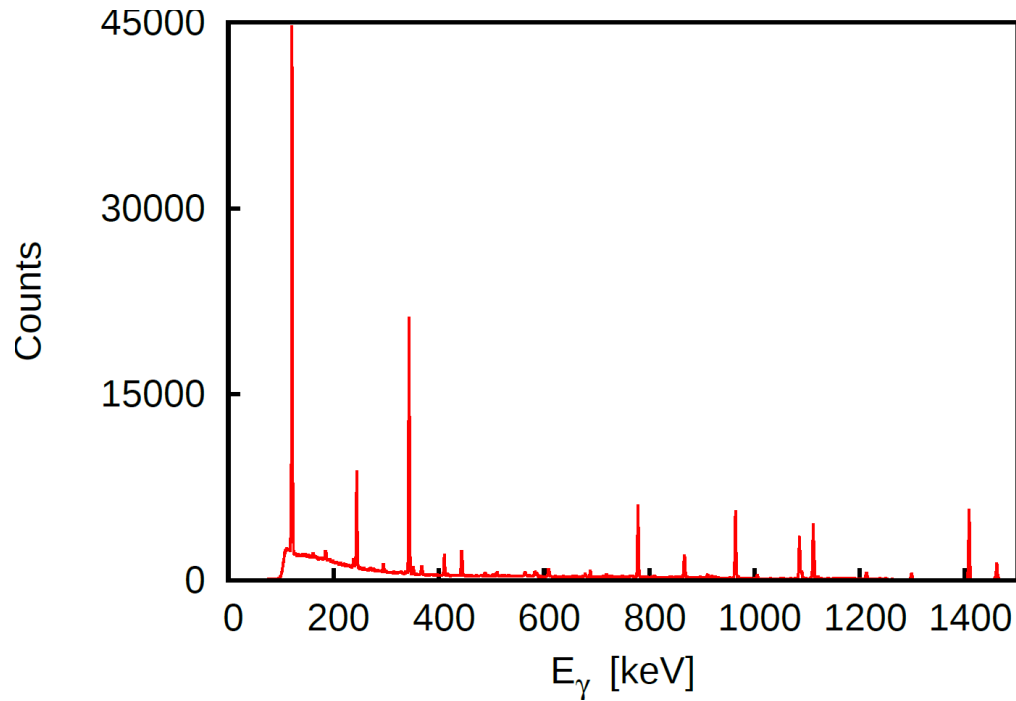
➤ Differential γ -production cross sections

$$\frac{d\sigma}{d\Omega}(\theta_i, E_k) = \frac{1}{4\pi} \frac{Y_j(E_k)}{Y_{FC}(E_k)} \frac{\varepsilon_{FC} \sigma_U(E_k)}{\varepsilon_j} \frac{t_U A_S}{t_S A_U} \frac{1}{c_{ms}(E_k)}$$

➤ The efficiency of the HPGe detectors

Two important steps:

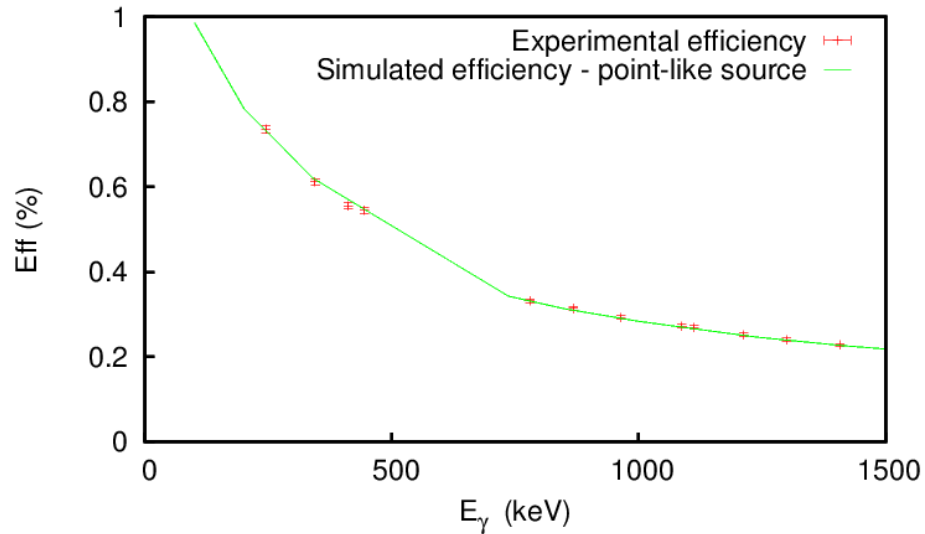
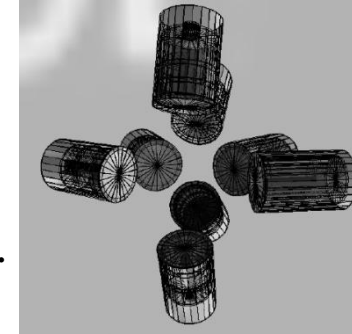
1. Experimental efficiencies using a ^{152}Eu point-like source.
2. Real efficiencies which take into account our extended sample (using MCNP6).



➤ The efficiency of the HPGe detectors

Two important steps:

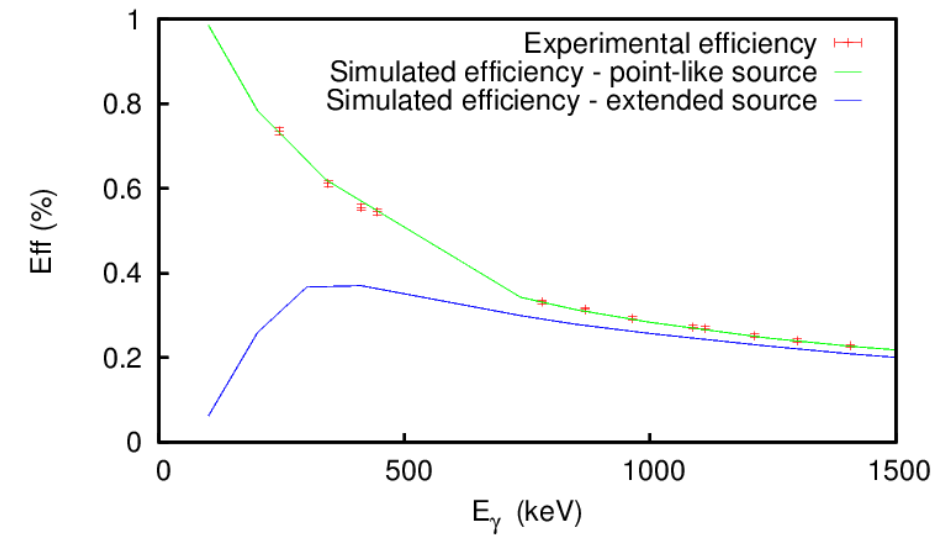
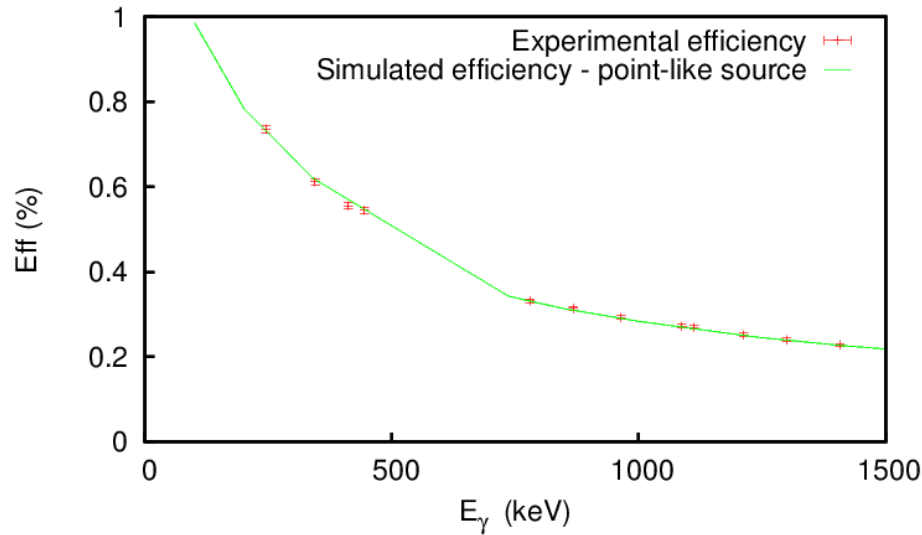
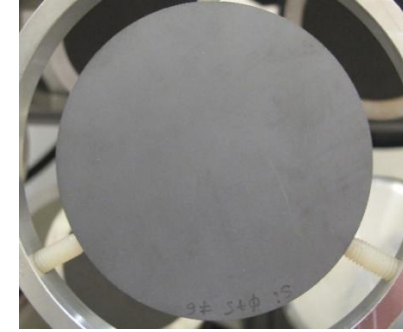
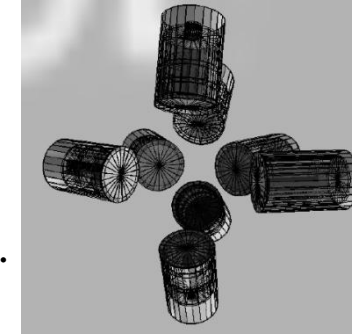
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➤ The efficiency of the HPGe detectors

Two important steps:

1. Experimental efficiencies using a ^{152}Eu point-like source.
2. Real efficiencies which take into account our extended sample (using MCNP6).



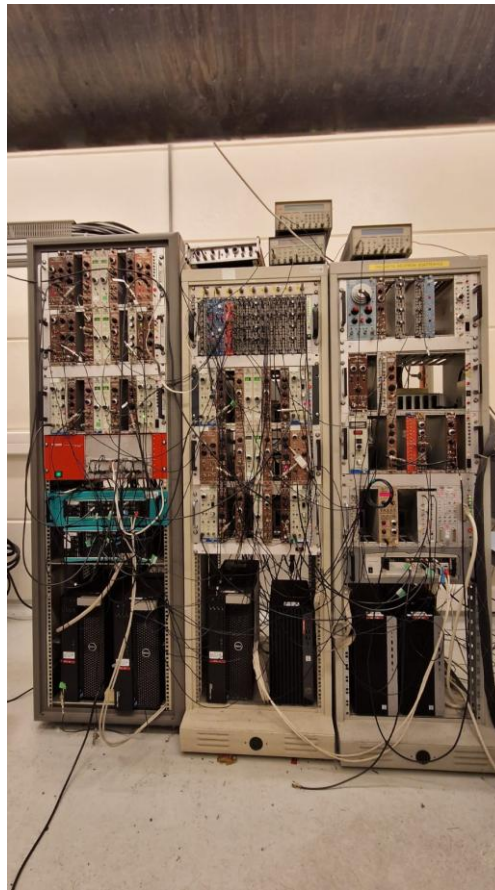
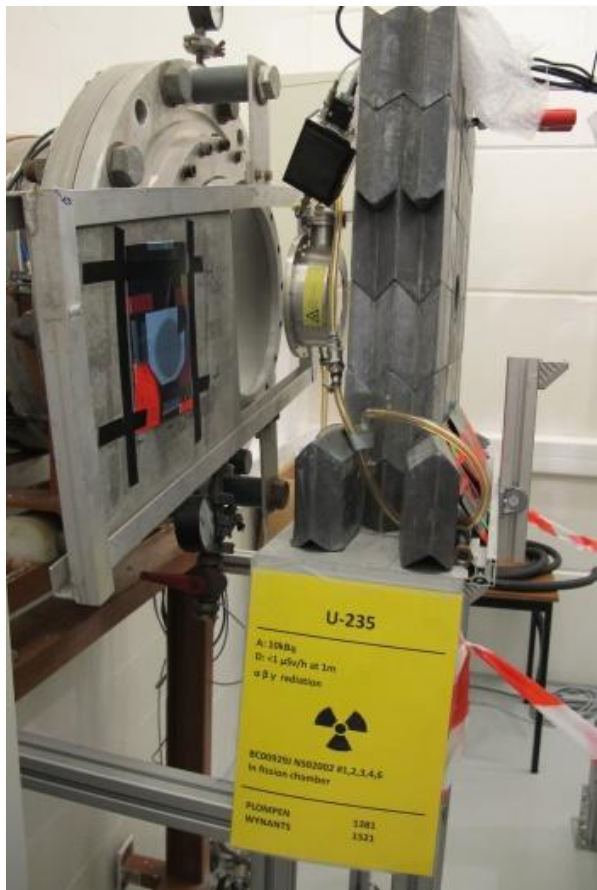
Data analysis procedure

Fission chamber data reduction



➤ Differential γ -production cross sections

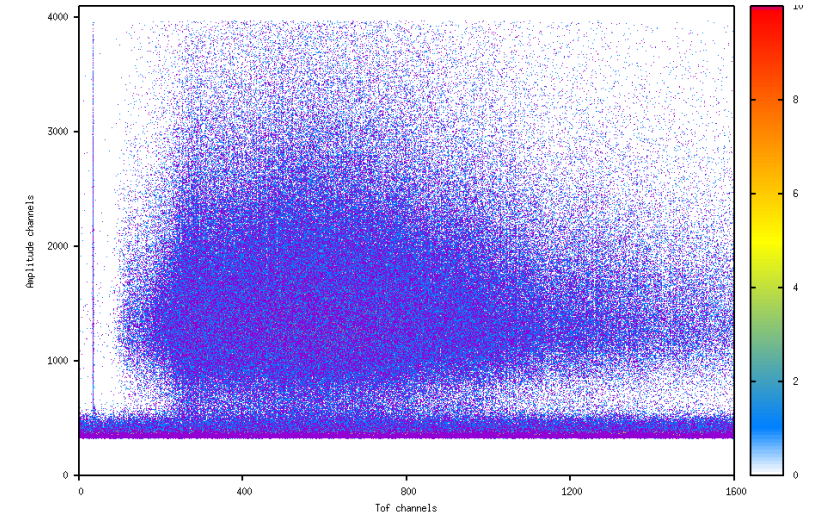
$$\frac{d\sigma}{d\Omega}(\theta_i, E_k) = \frac{1}{4\pi} \frac{Y_j(E_k)}{Y_{FC}(E_k)} \frac{\varepsilon_{FC} \sigma_U(E_k)}{\varepsilon_j} \frac{t_U A_S}{t_S A_U} \frac{1}{c_{ms}(E_k)}$$



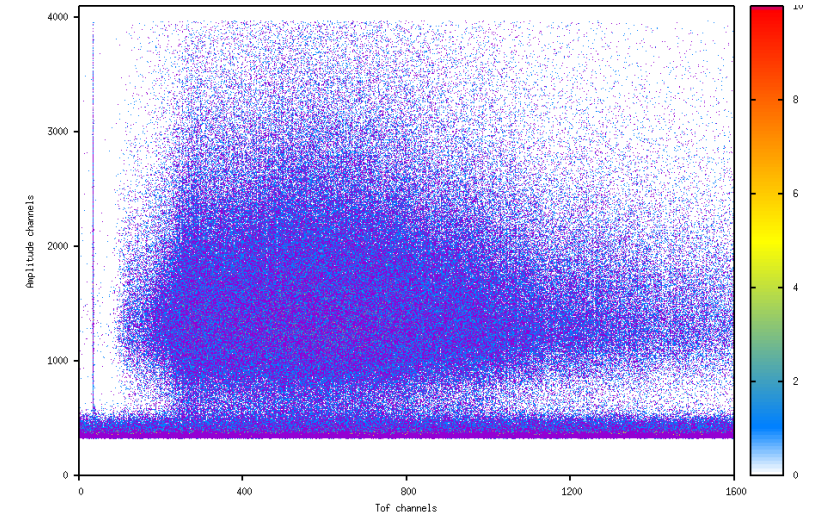
➤ Time-amplitude listfiles recorded for 2h



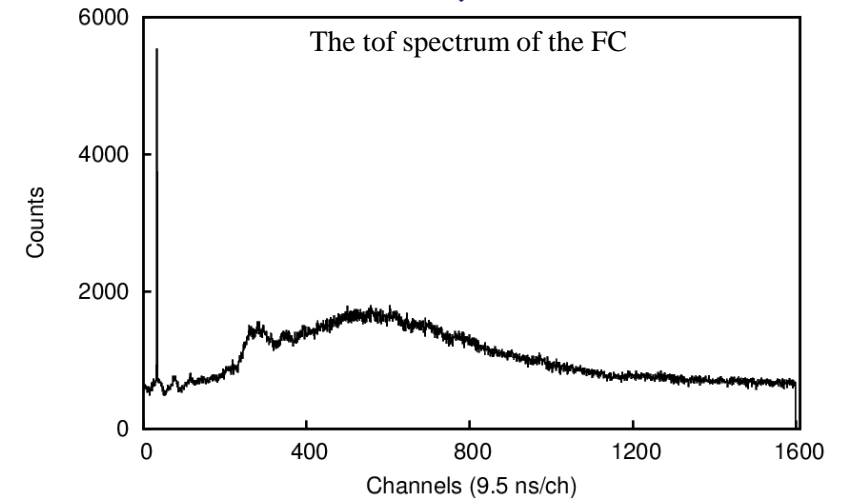
The time-amplitude matrix of the FC



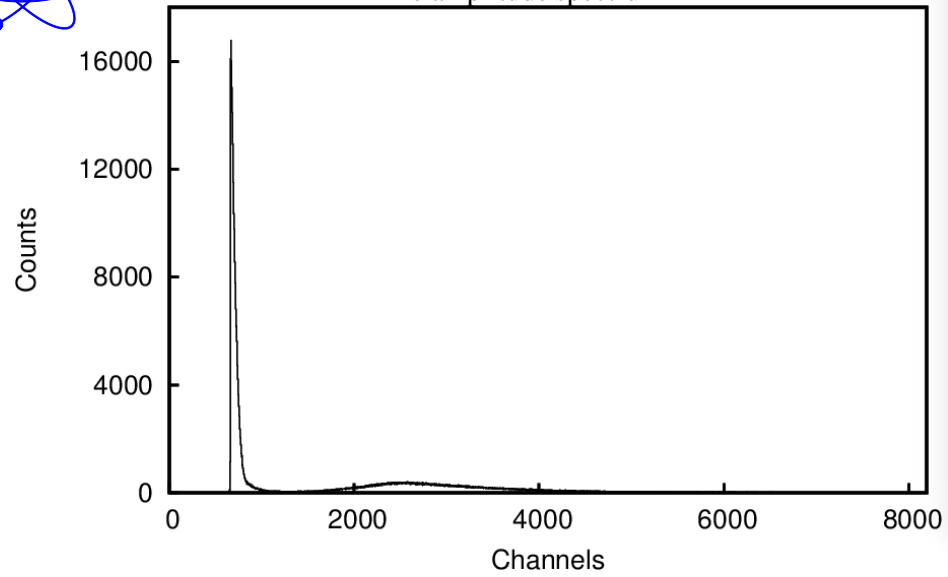
The time-amplitude matrix of the FC



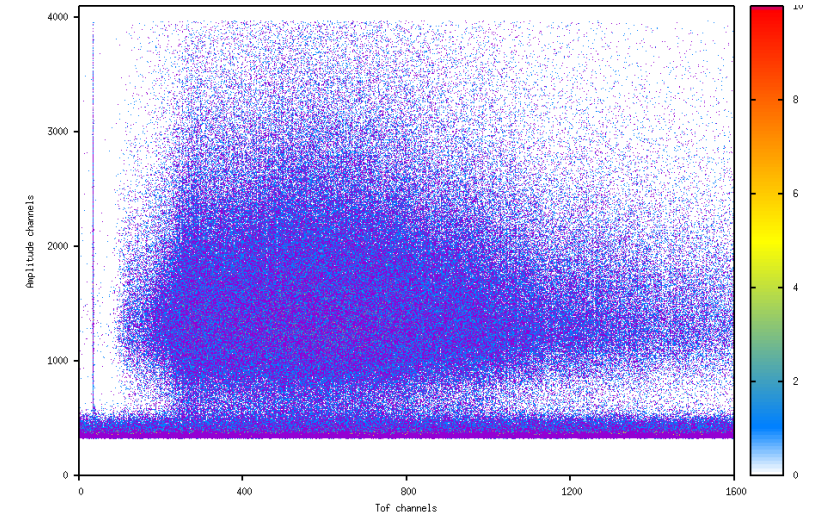
The tof spectrum of the FC



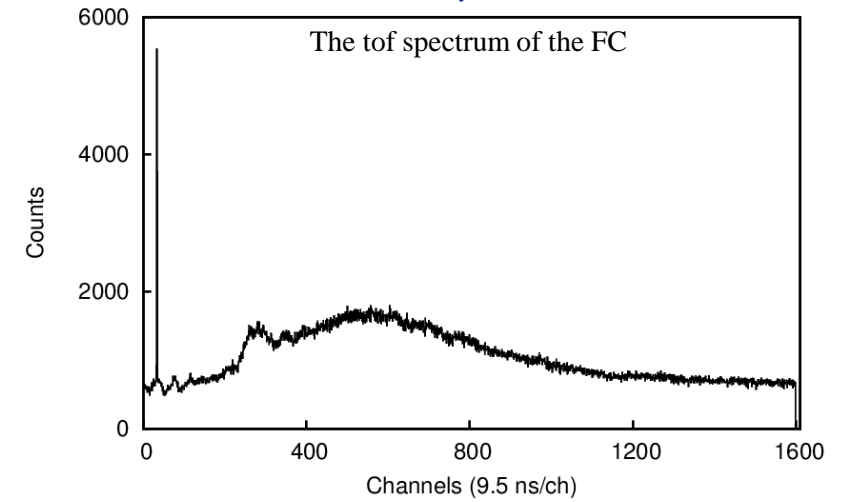
The amplitude spectrum



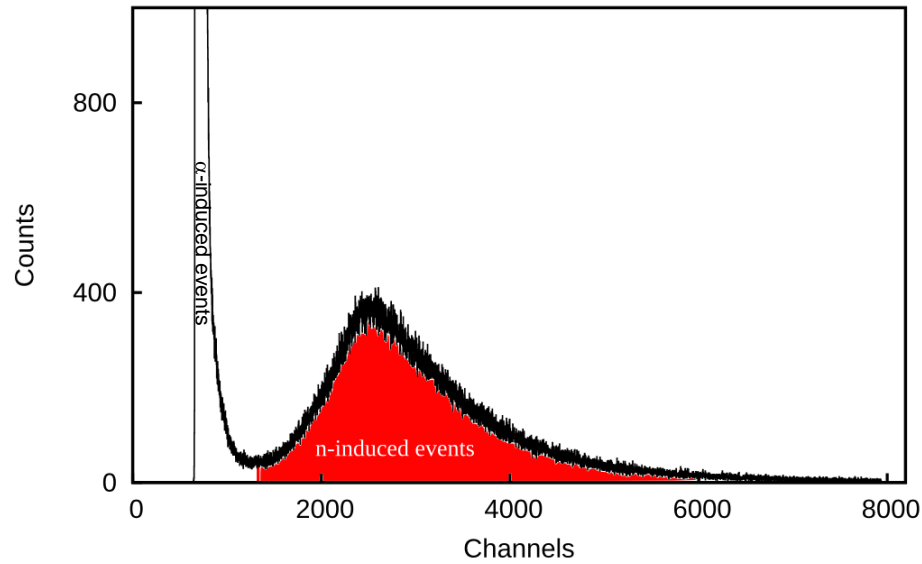
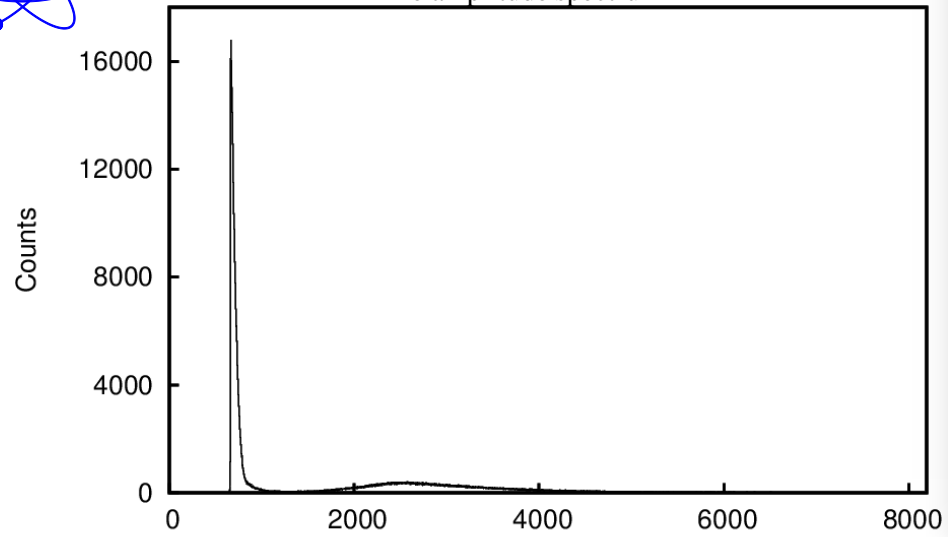
The time-amplitude matrix of the FC



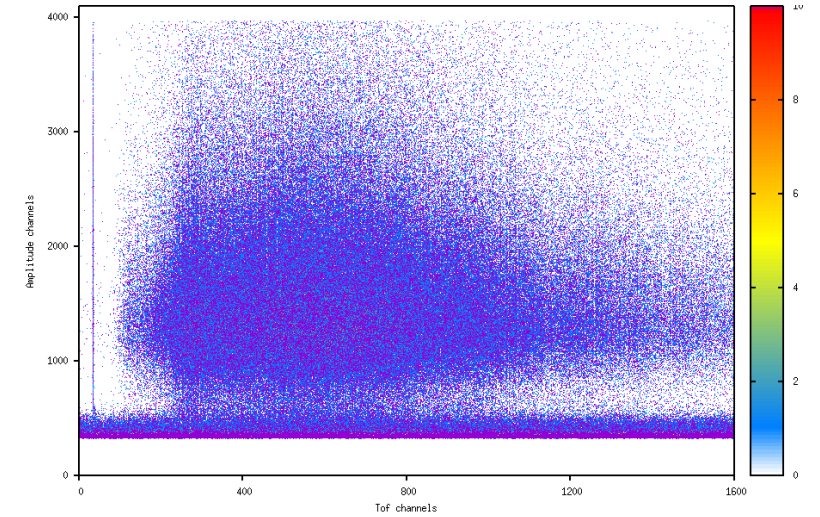
The tof spectrum of the FC



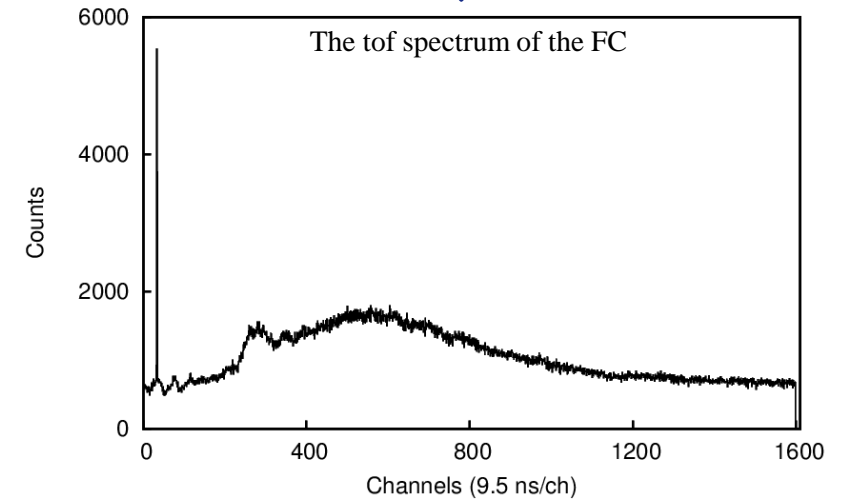
The amplitude spectrum

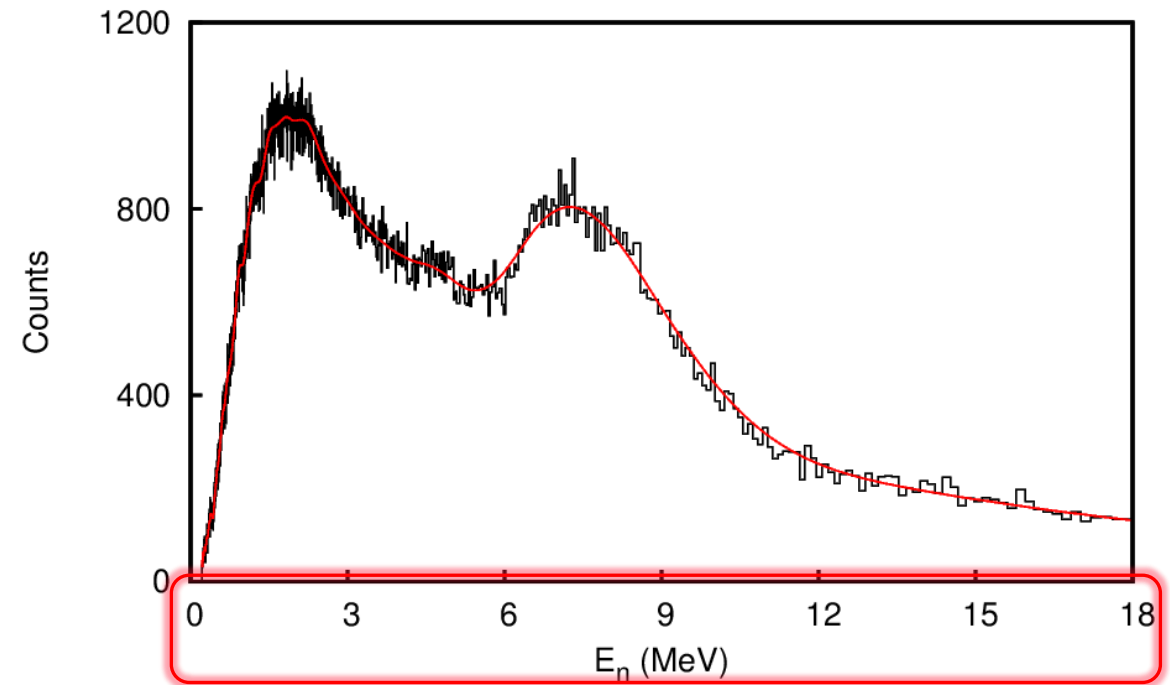
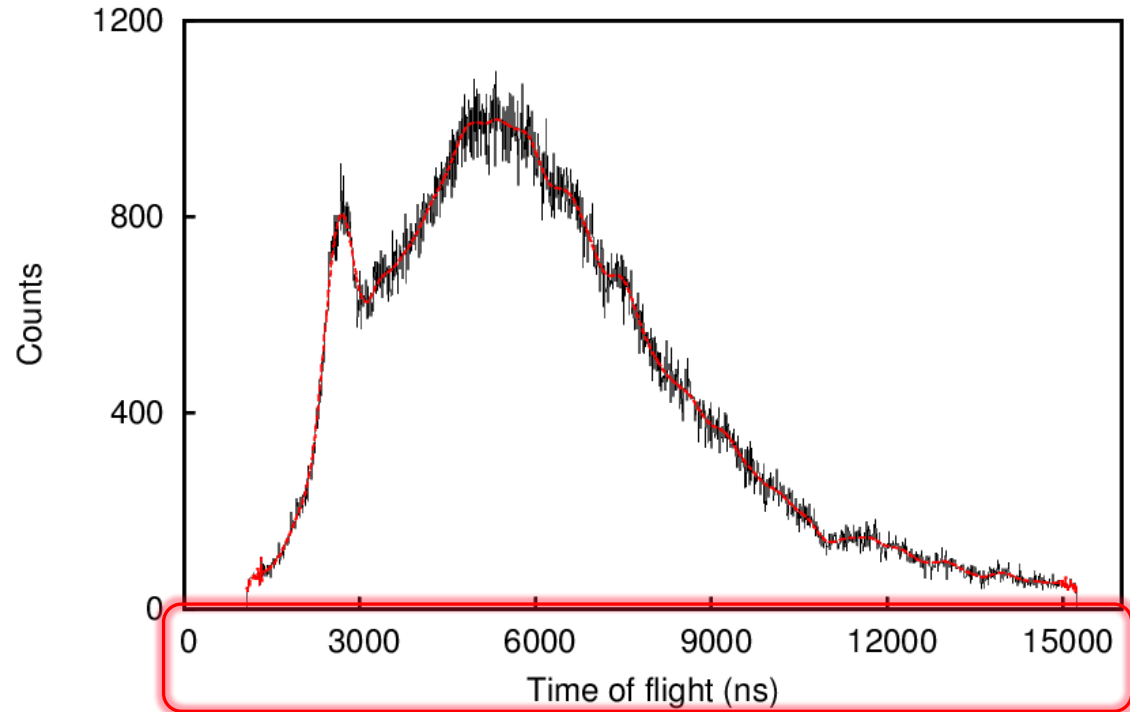


The time-amplitude matrix of the FC



The tof spectrum of the FC

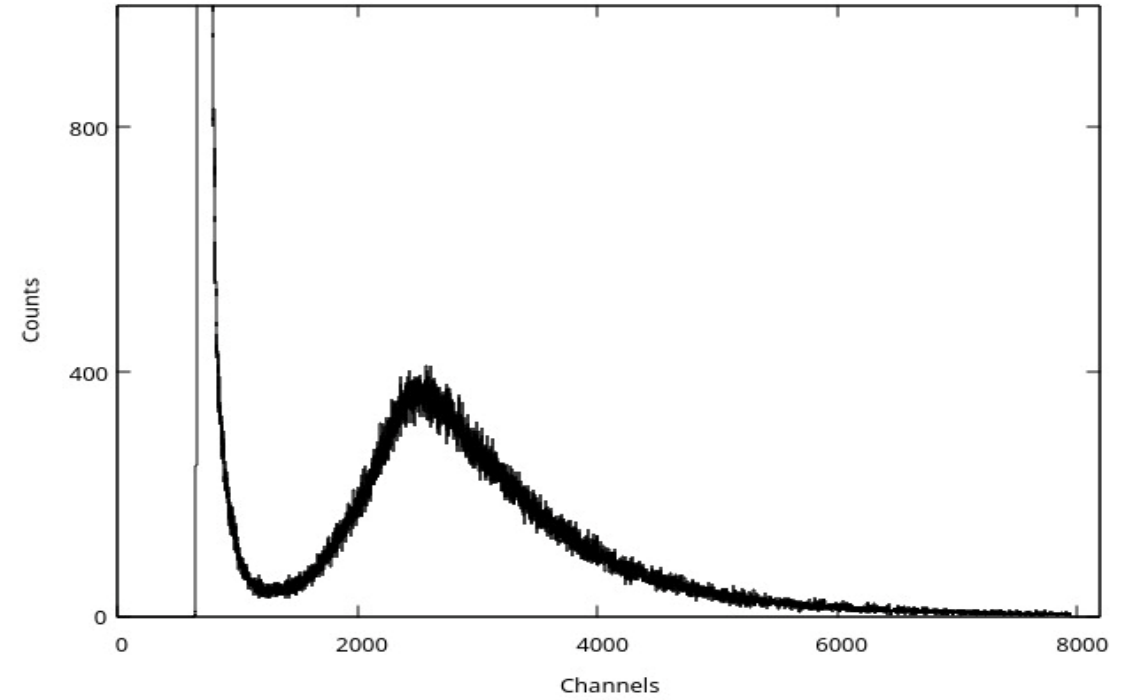
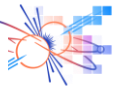




➤ Differential γ -production cross sections

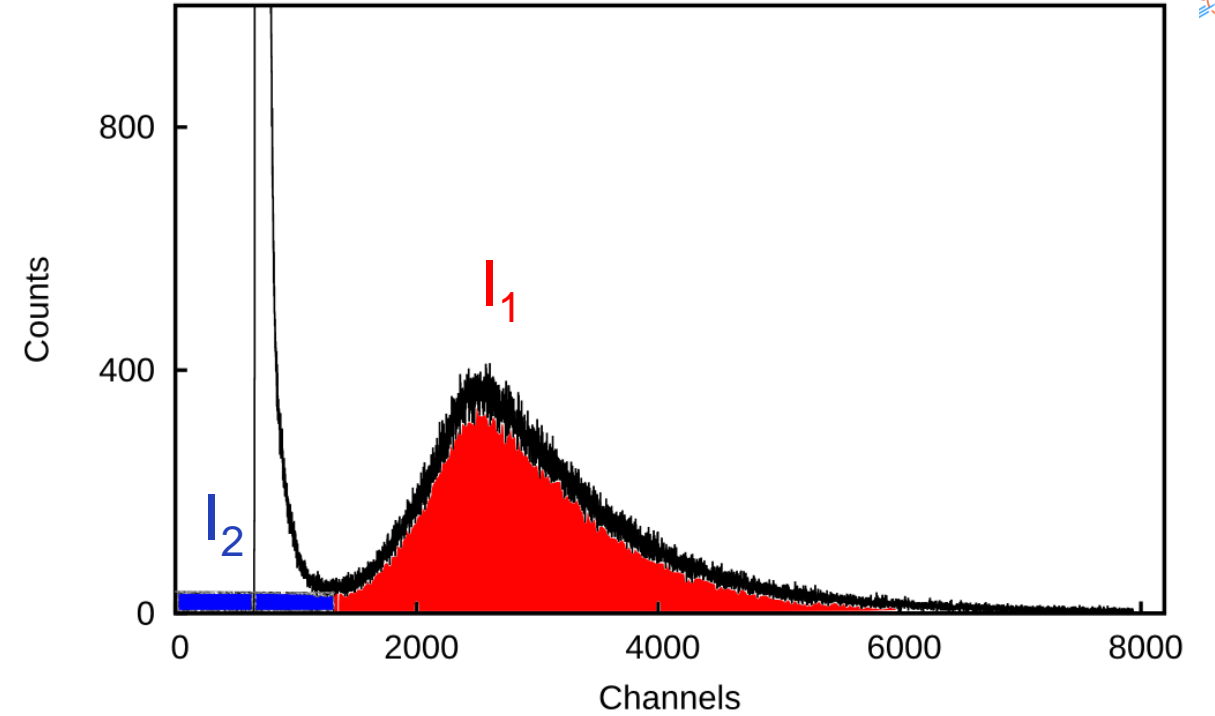
$$\frac{d\sigma}{d\Omega}(\theta_i, E_k) = \frac{1}{4\pi} \frac{Y_j(E_k)}{Y_{FC}(E_k)} \frac{\varepsilon_{FC}}{\varepsilon_j} \frac{\sigma_U(E_k)}{t_s} \frac{A_S}{A_U} \frac{1}{c_{ms}(E_k)}$$

➤ The efficiency of the fission chamber



➤ The efficiency of the fission chamber

$$\varepsilon_1 = \frac{I_1}{I_1 + I_2} = 0.920^*$$

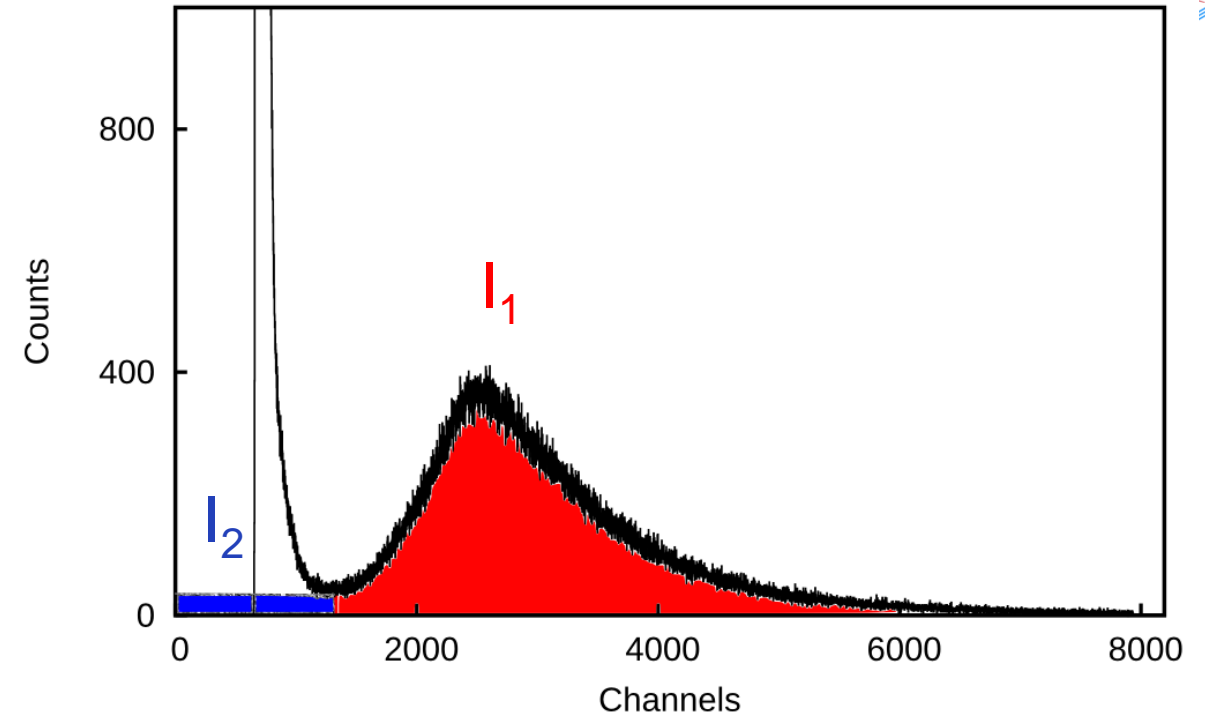


* experimental value

➤ The efficiency of the fission chamber

$$\varepsilon_1 = \frac{I_1}{I_1 + I_2} = 0.920^*$$

$$\varepsilon_2 = 0.982^{**}$$



* experimental value

** accounts for the number of the fission fragments that stop in the deposits

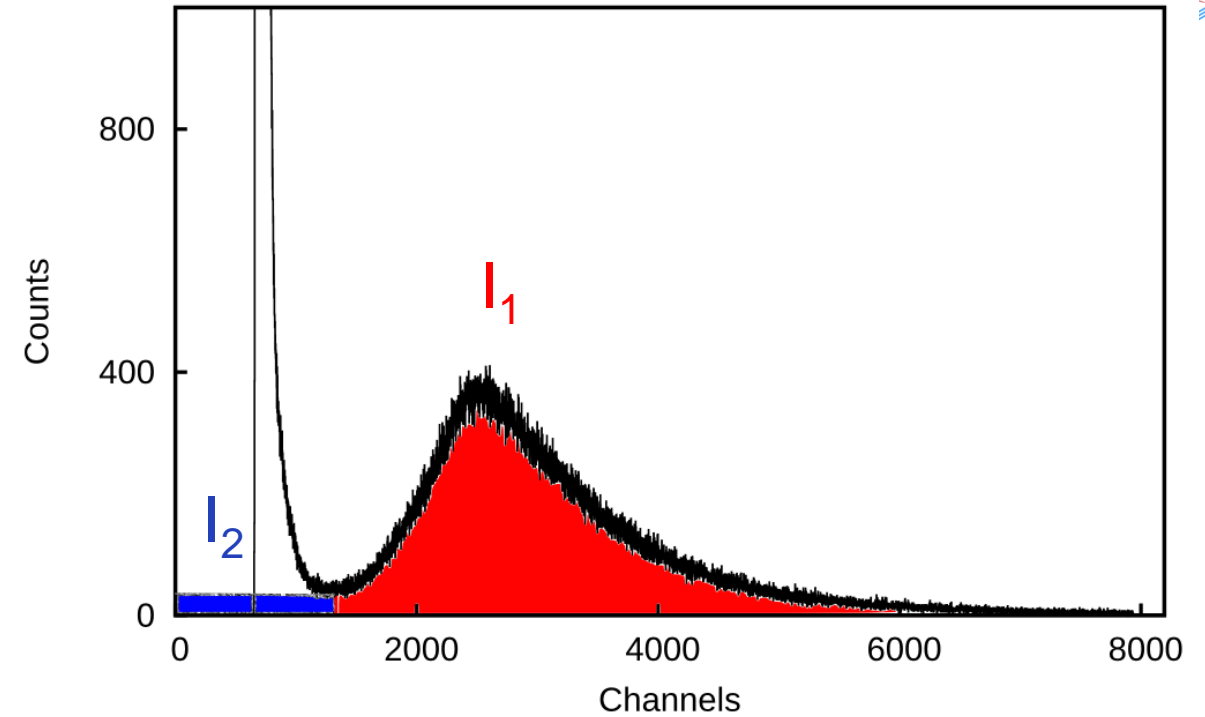
➤ The efficiency of the fission chamber

$$\varepsilon_1 = \frac{I_1}{I_1 + I_2} = 0.920^*$$

$$\varepsilon_2 = 0.982^{**}$$

$$\varepsilon_3 = 0.947^{***}$$

$$\varepsilon_{FC} = \varepsilon_1 \cdot \varepsilon_2 \cdot \varepsilon_3 = 0.855$$



* experimental value

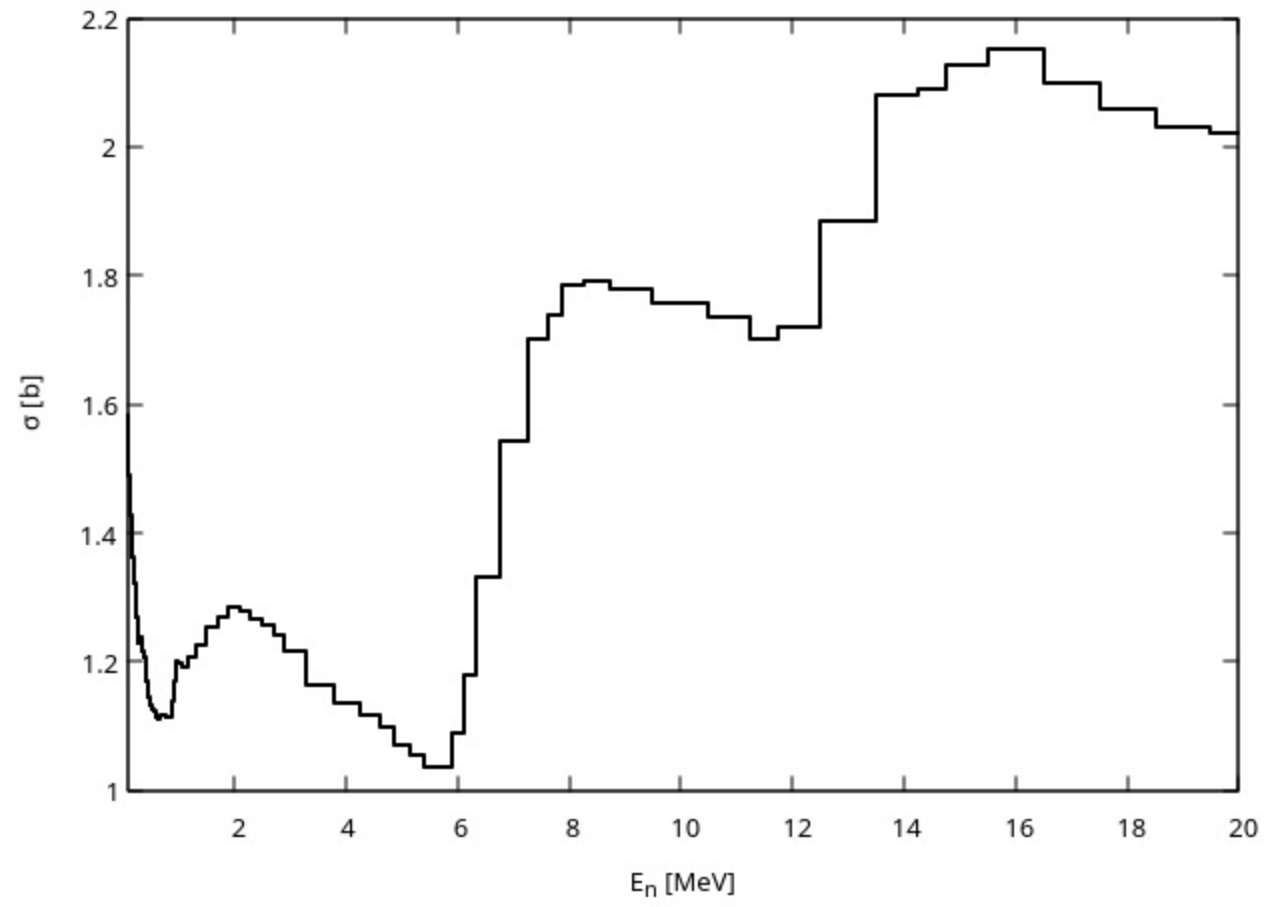
** accounts for the number of the fission fragments that stop in the deposits

*** accounts for the inhomogeneity of the UF₄ deposits

➤ Differential γ -production cross sections

$$\frac{d\sigma}{d\Omega}(\theta_i, E_k) = \frac{1}{4\pi} \frac{Y_j(E_k)}{Y_{FC}(E_k)} \frac{\varepsilon_{FC} \sigma_U(E_k)}{\varepsilon_j} \frac{t_U}{t_S} \frac{A_S}{A_U} \frac{1}{c_{ms}(E_k)}$$

➤ The standard $^{235}\text{U}(n,f)$ cross section used as a reference



➤ Differential γ -production cross sections

$$\frac{d\sigma}{d\Omega}(\theta_i, E_k) = \frac{1}{4\pi} \frac{Y_j(E_k)}{Y_{FC}(E_k)} \frac{\varepsilon_{FC} \sigma_U(E_k)}{\varepsilon_j} \frac{t_U A_S}{t_S A_U} \frac{1}{c_{ms}(E_k)}$$

➤ Constants:

- Areal density of the ^{235}U deposit ($t_U=3.089 \times 10^{-3} \text{ g/cm}^2$)
- Areal density of the sample ($t_S= 0.933 \text{ g/cm}^2$)
- Atomic mass of ^{235}U ($A_U=235.043 \text{ a.m.u}$)
- Atomic mass of sample ($A_S=53.939 \text{ a.m.u}$)

➤ Differential γ -production cross sections

$$\frac{d\sigma}{d\Omega}(\theta_i, E_k) = \frac{1}{4\pi} \frac{Y_j(E_k)}{Y_{FC}(E_k)} \frac{\varepsilon_{FC} \sigma_U(E_k)}{\varepsilon_j} \frac{t_U A_S}{t_S A_U} \frac{1}{c_{ms}(E_k)}$$

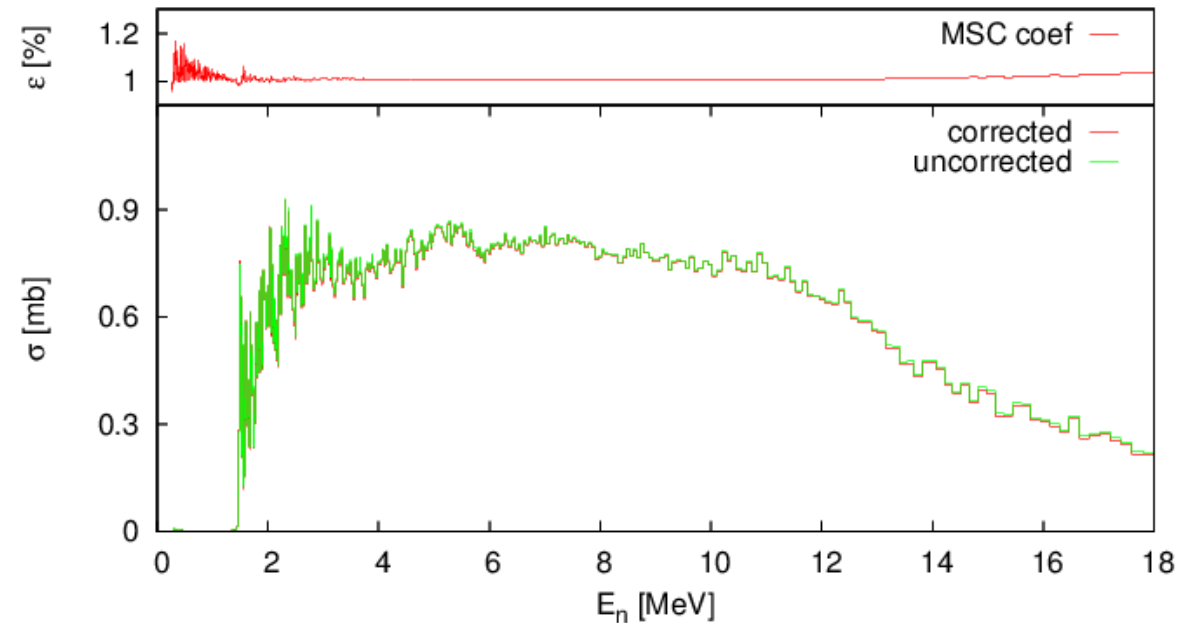
➤ Neutron multiple scattering and attenuation corrections (using MCNP6)

- take into account the effects in the sample and in the surrounding materials.
- important because if a neutron undergoes multiple scattering or is attenuated the flux is altered and also the time of the (n, n'γ) event will not correspond to the energy of the incident neutron.

- simultaneously taken into account via MCNP 6 simulations

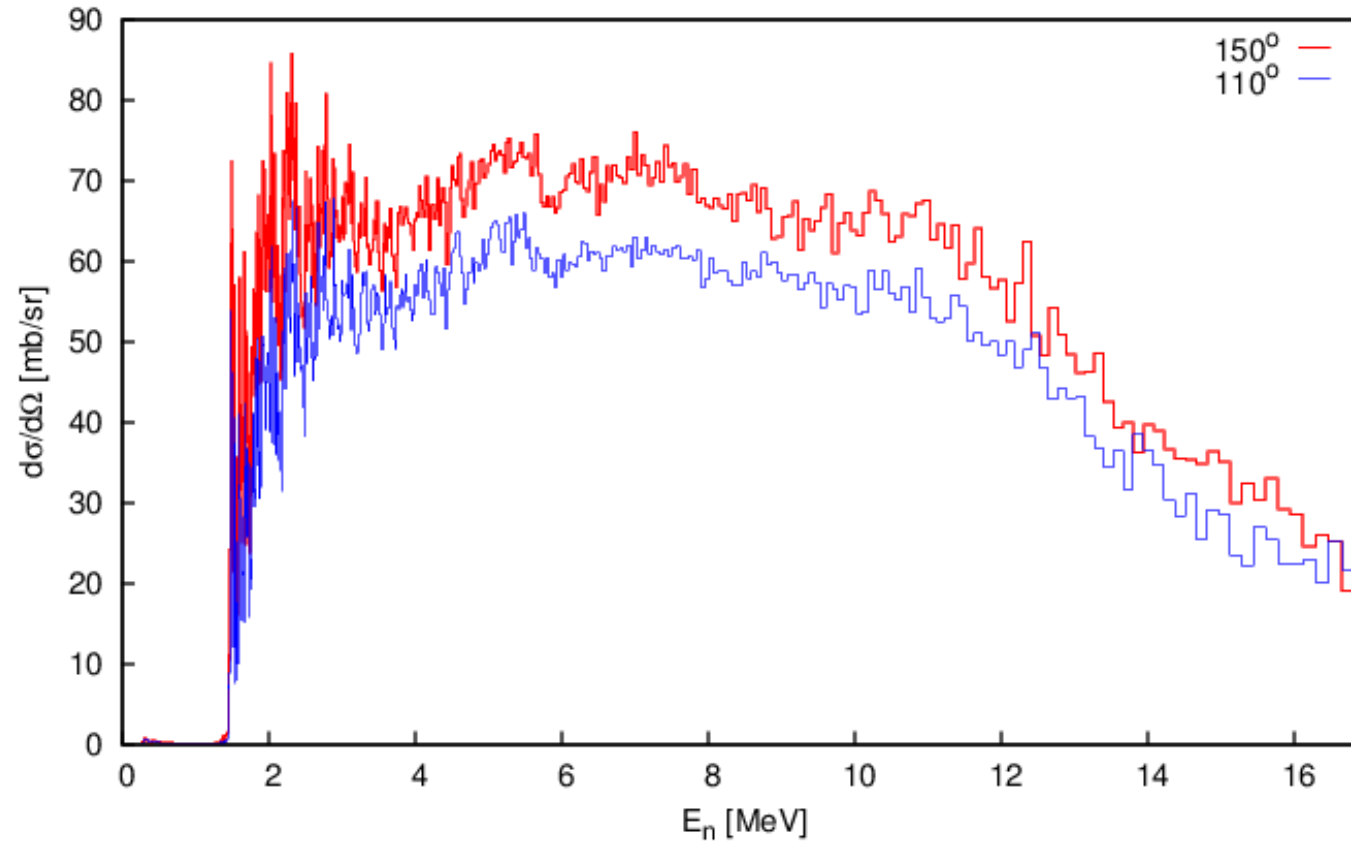
Steps:

1. Geometry description
2. Measured γ -production cross section used as a dose card
3. Reaction rates calculated both for full geometry and for void after fission chamber
4. Correction factor determined by the ratio of the two reaction rates for each γ -production cross section



➤ Differential γ -production cross sections

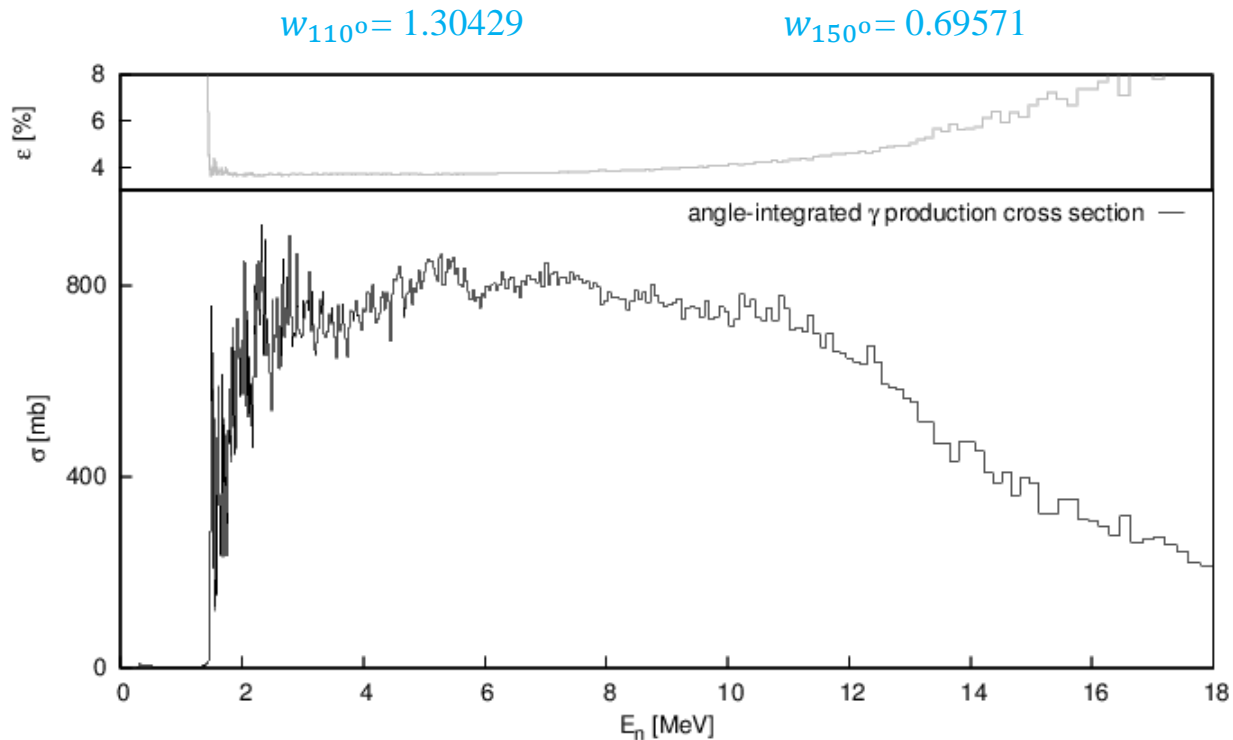
$$\frac{d\sigma}{d\Omega}(\theta_i, E_k) = \frac{1}{4\pi} \frac{Y_j(E_k)}{Y_{FC}(E_k)} \frac{\varepsilon_{FC} \sigma_U(E_k)}{\varepsilon_j} \frac{t_U}{t_s} \frac{A_S}{A_U} \frac{1}{c_{ms}(E_k)}$$

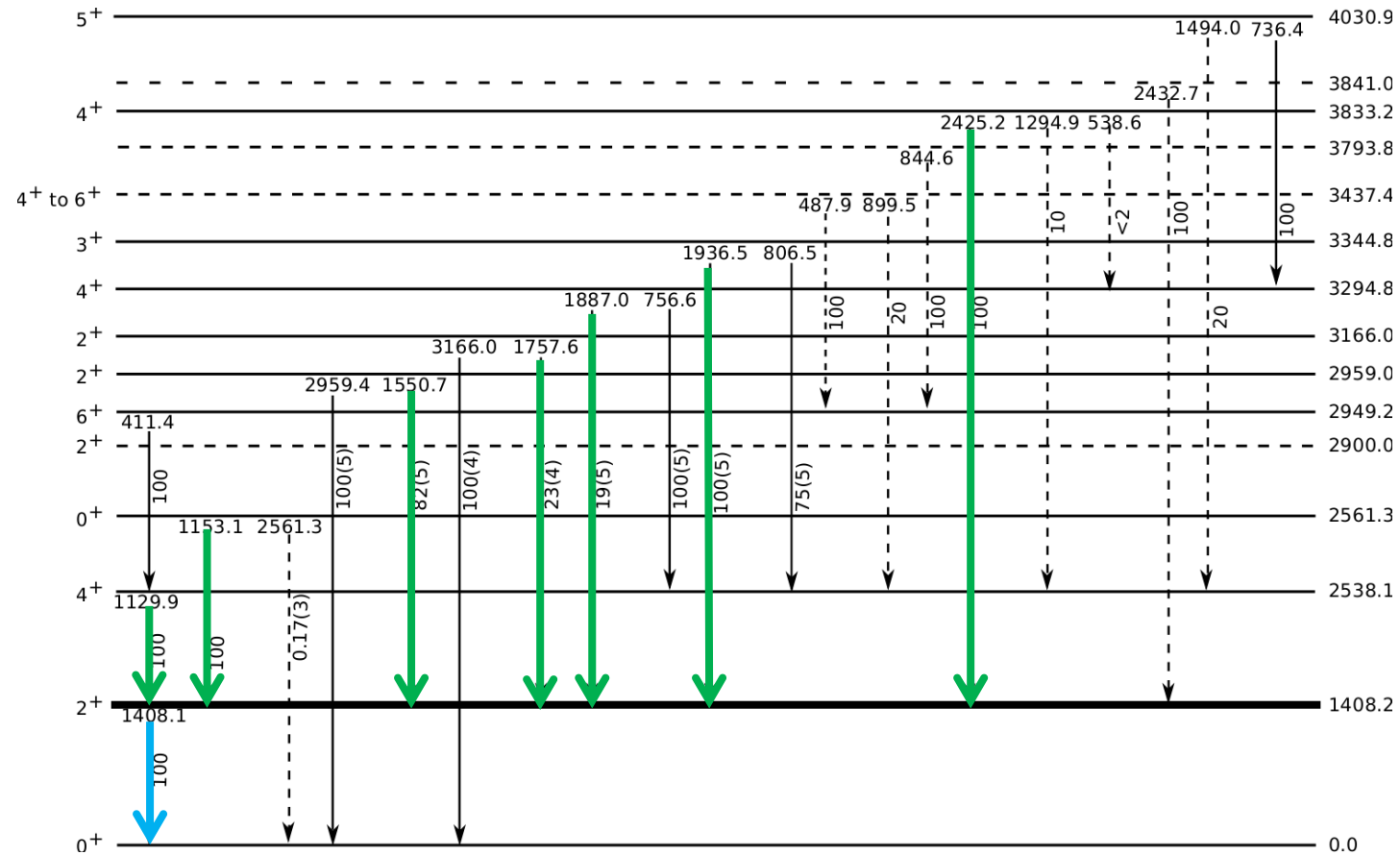


➤ Angle-integrated γ -production cross sections

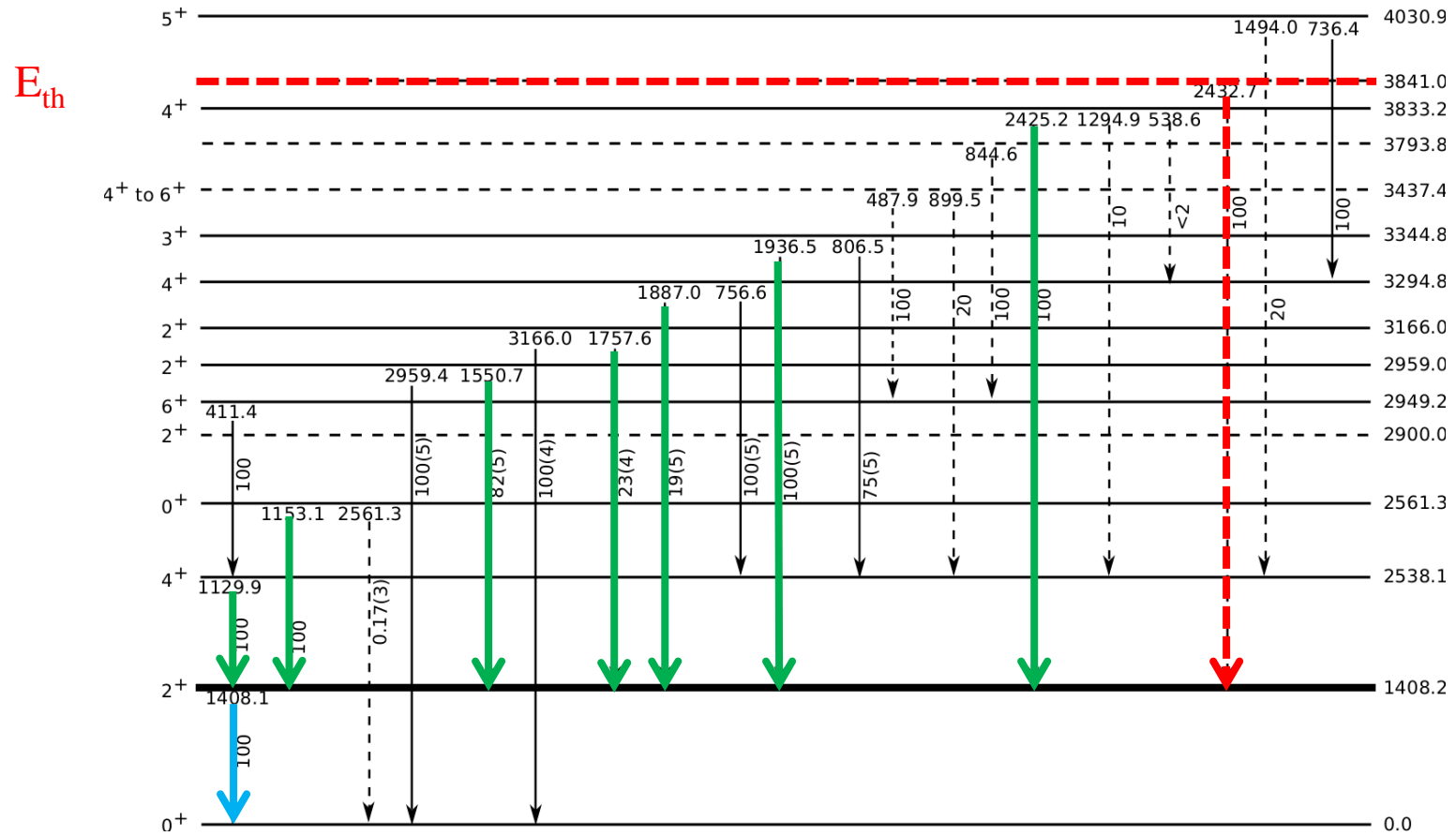
- Calculated using the gaussian quadrature method
- Writing the differential cross sections in terms of Legendre polynomials
- $\cos(110^\circ)$ and $\cos(150^\circ)$ are nodes of the 4th order Legendre polynomial

$$\sigma(E_k) = 2\pi [w_{110^\circ} \frac{d\sigma}{d\Omega}(110^\circ, E_k) + [w_{150^\circ} \frac{d\sigma}{d\Omega}(150^\circ, E_k)]]$$

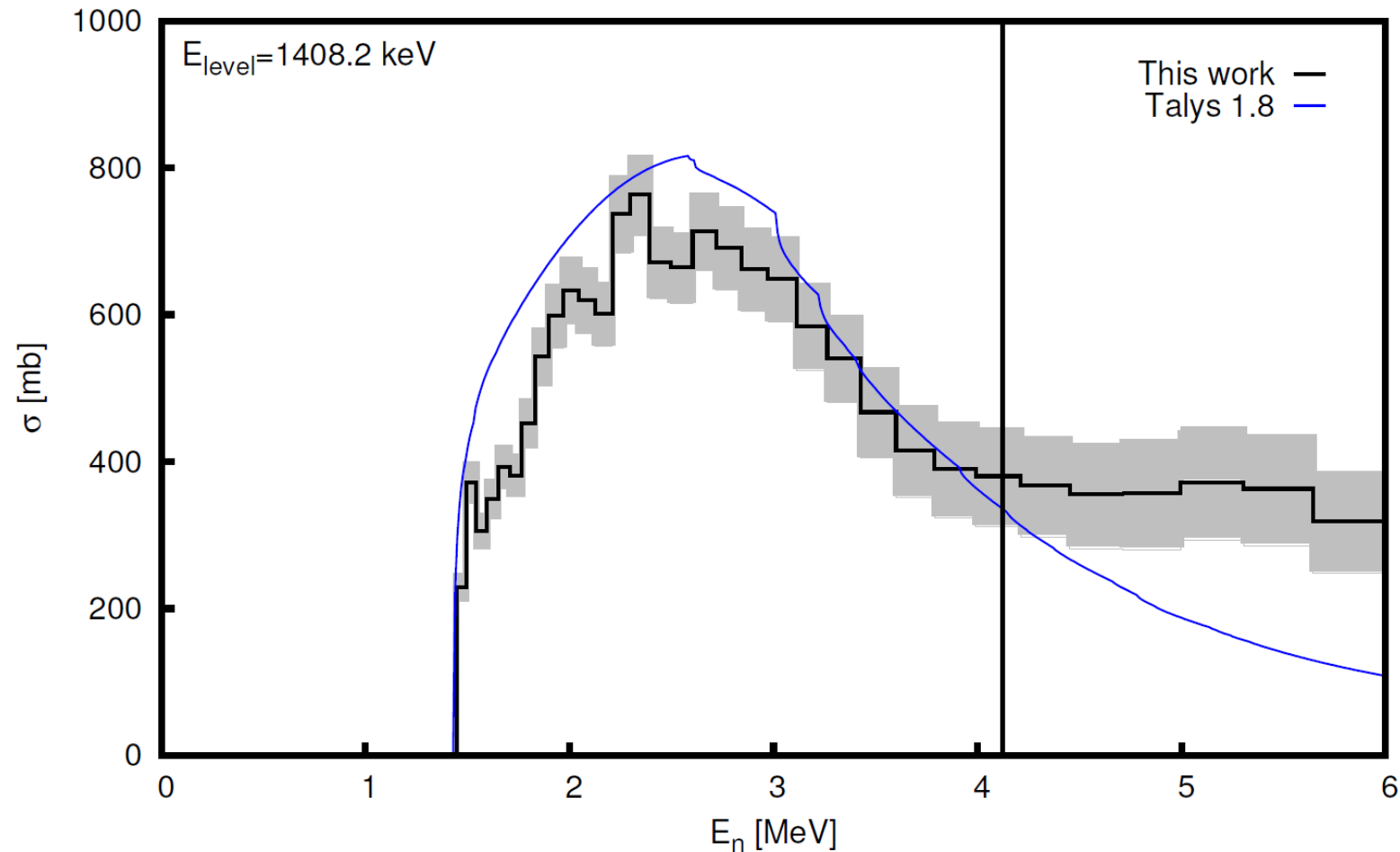


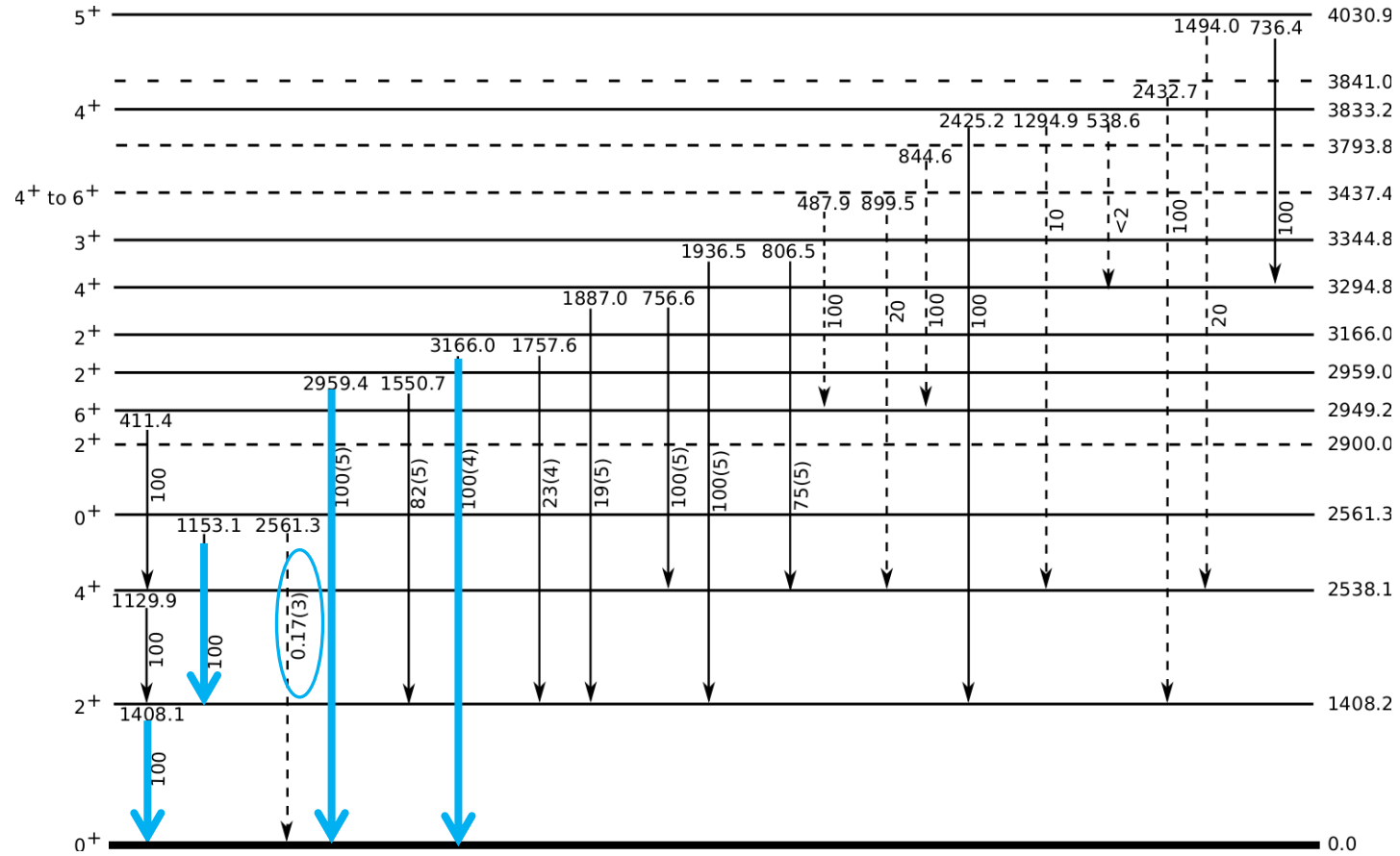


- Limited by our detection possibility.



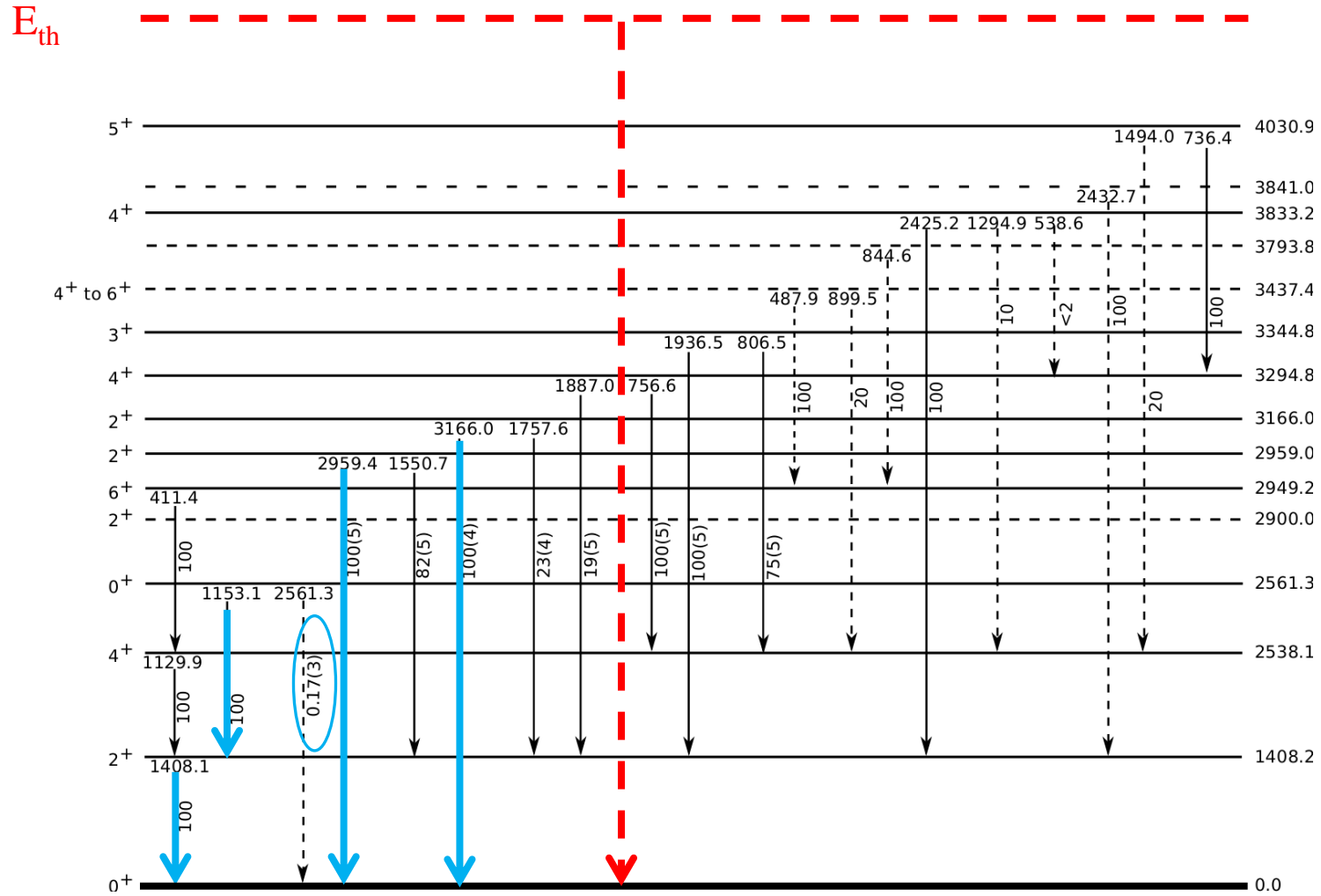
- Limited by our detection possibility.

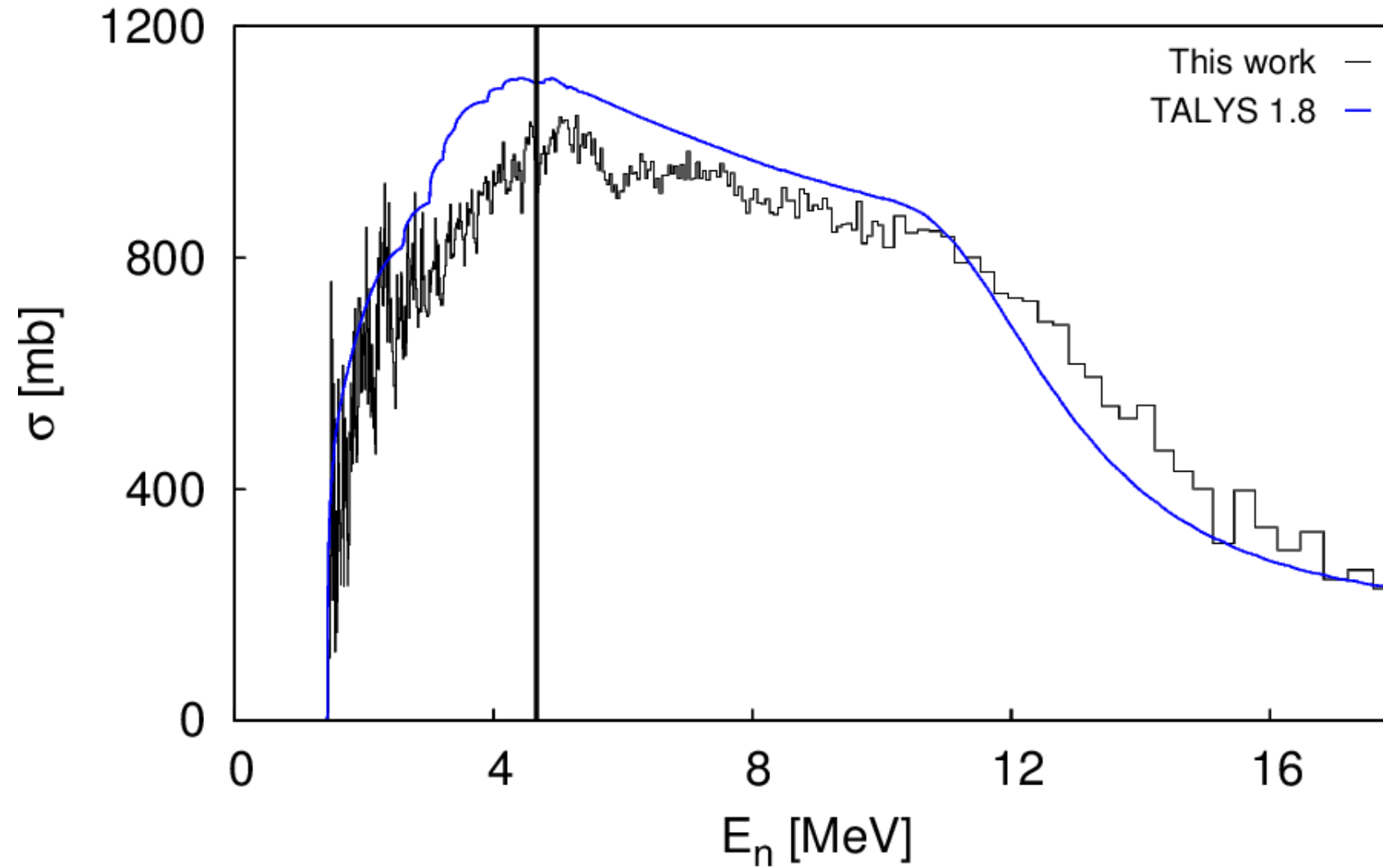




➤ Total inelastic cross sections

- Limited by our detection possibility.





Data analysis procedure

Uncertainty calculation



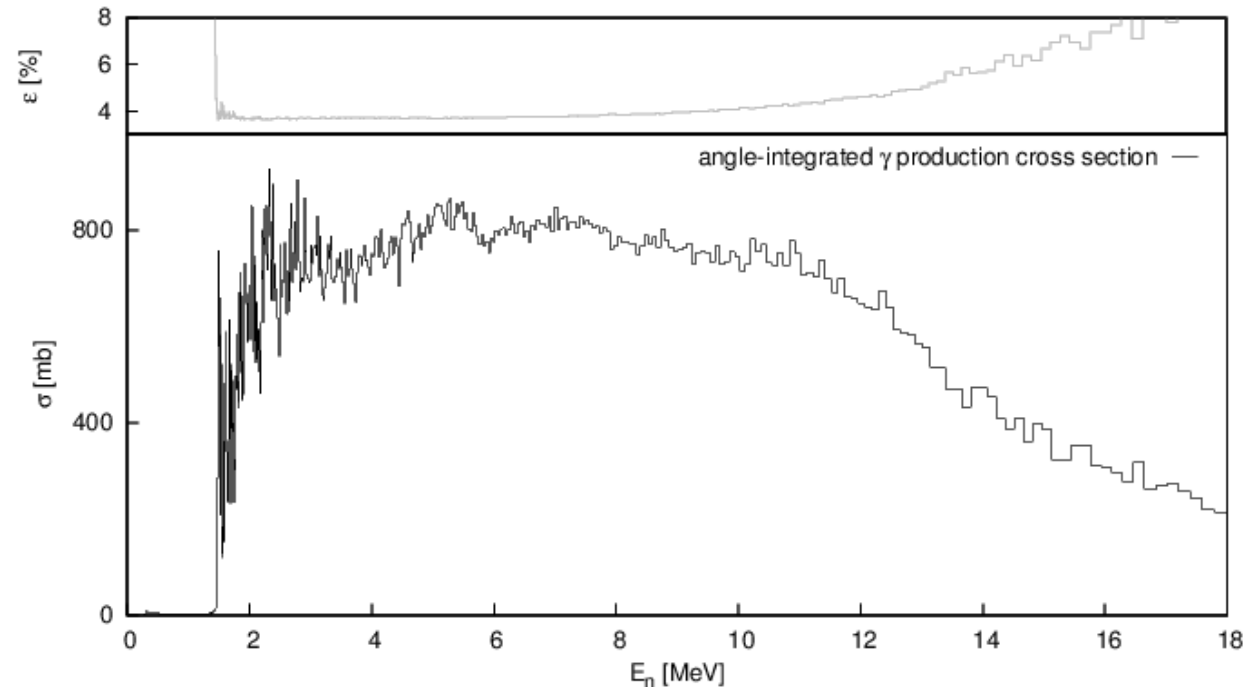
➤ Differential γ -production cross sections: primary results

$$\frac{d\sigma}{d\Omega}(\theta_i, E_k) = \frac{1}{4\pi} \frac{Y_j(E_k)}{Y_{FC}(E_k)} \frac{\varepsilon_{FC} \sigma_U(E_k)}{\varepsilon_j} \frac{t_U A_S}{t_S A_U} \frac{1}{c_{ms}(E_k)}$$

➤ Angle-integrated γ -production cross sections: main reported results

$$\sigma(E_k) = 2\pi [w_{110^\circ} \frac{d\sigma}{d\Omega}(110^\circ, E_k) + [w_{150^\circ} \frac{d\sigma}{d\Omega}(150^\circ, E_k)]]$$

Parameter	Value (%)
$Y_j(E_k)$	$\cong 1\%$ ($E_n < 10$ MeV) $\cong 5\%$ ($E_n = 10 \div 20$ MeV)
$Y_{FC}(E_k)$	3%
ε_{FC}	2%
ε_j	2%
$\sigma_U(E_k)$	<1%
$c_{ms}(E_k)$	<1%
t_U	<1%
t_S	2%



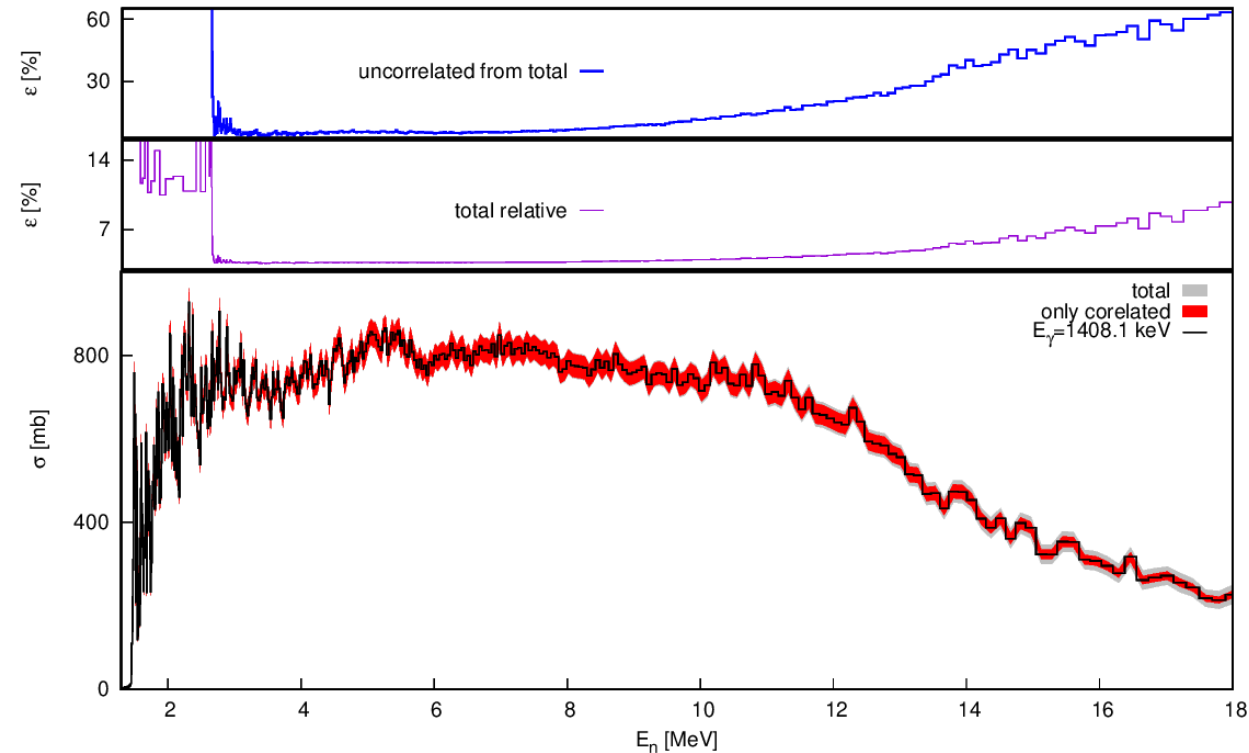
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ε_{FC}	2%
ε_j	2%
$\sigma_U(E_k)$	<1%
$c_{ms}(E_k)$	<1%
t_U	<1%
t_S	2%





Get in touch for more information:



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adina.olacel@nipne.ro

Thank you!



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